## MHD Simulation of Multi-pulsed Helicity Injection in High-q ST Plasmas

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# **Helicity-driven spherical system**



Many experiments on the **coaxial helicity injection** (CHI) using a magnetized coaxial plasma gun (MCPG) have been carried out for **spherical torus** (ST) to understand the mechanism of current drive or MHD relaxation.



-- Formation and sustainment of the ST configuration by CHI are considered to be due to an n=1 kink mode driven by the OFC.

-- This mode converts injected toroidal flux into poloidal flux, resulting in an **flux amplification** which involves **magnetic reconnection events**. In the driven phase, the events open magnetic surfaces and deteriorate **energy confinement**.

-- In the decay phase after the driven phase, the magnetic fluctuations become small, allowing closed flux surfaces to form, resulting in good confinement.

# Multi-pulsed CHI (M-CHI) scheme



Driven phase: Current drive is performed with allowing the deterioration of confinement.
Decay phase: Closed flux surfaces are formed, resulting in good confinement.

The M-CHI scenario aims to achieve simultaneously a **quasi-steady sustainment** and **good confinement** by repeating the driven and decay phases.

The purpose of this study is to perform the 3-D nonlinear MHD simulations to investigate the dynamics of the magnetic field structure and plasma flows generated during the M-CHI in the high-q ST (q>1).











### HIST plameters



• TF coil current

**Spheromak, Low-q ST:**  $q \sim I_{tf} (= 0.30 \text{ kA}) / I_t < 1$ **High-q ST:**  $q \sim I_{tf} (= \sim 150 \text{ kA}) / I_t > 1$ 

- Power supply system for double-pulse
  - Formation capacitor banks V = 3-10 kV, C = 0.6 mFInjection current :  $I_g \sim 30 - 60 \text{ kA}$
- Sustainment capacitor banks First pulse : *V* < 900 V, *C* = 336 mF Second pulse : *V* < 900 V, *C* = 195 mF

2<sup>nd</sup> pulse voltage:  $V_g \sim 400 V$ 2<sup>nd</sup> pulse current:  $I_g \sim 10-20 \text{ kA}$ 

# **Double pulsing CHI discharge**





By secondly pulsing the MCPG at t = 1.5 or 2.5 ms during the partially decay phase, total plasma current is effectively amplified against the resistive decay. The core current density is generated due to dynamo.

(High-q)

- The sustainment time has increased up to 6 -8 ms which is longer than that in the single CHI case.
- **\*** The edge  $\lambda$  in the OFC is larger than the core  $\lambda$ , causing helicity transport.

 $\lambda = \mu_0 I_{\rm t.} / \Psi_{\rm t}$ 

Ion Doppler temperature increases from 20 eV up to 30 eV.

# Flows and density profiles







# Radial profile of dynamo electric field





- Hall dynamo driven current in the OFC is the same direction as the mean current.
- MHD anti-dynamo electric field in the OFC reduces the mean current.



[1] T.R. Jarboe *et al.*, Nucl. Fusion **51** 063029 (2011).



$$\frac{\partial \rho \mathbf{v}}{\partial t} = -\nabla \cdot \rho \mathbf{v} \mathbf{v} + \mathbf{j} \times \mathbf{B} - \nabla p - \nabla \cdot \mathbf{\vec{\Pi}}$$

$$\frac{\partial \mathbf{B}}{\partial t} = -\nabla \times \mathbf{E}$$

1) 
$$\rho$$
: Spatially  
and temporally  
2) constant

$$\frac{\partial p}{\partial t} = -\nabla \cdot \left( p\mathbf{v} - \kappa \nabla T \right) - \left( \gamma - 1 \right) \left( p\nabla \cdot \mathbf{v} + \mathbf{\vec{\Pi}} : \nabla \mathbf{v} - \eta \, j^2 \right) \tag{3}$$

$$\mathbf{E} = -\mathbf{v} \times \mathbf{B} + \eta \,\mathbf{j} \tag{4}$$

$$\mathbf{j} = \nabla \times \mathbf{B} \tag{5}$$

$$T = p / \rho \tag{6}$$

$$\vec{\mathbf{\Pi}} = \nu \left( \frac{2}{3} (\nabla \cdot \mathbf{v}) \vec{\mathbf{I}} - \nabla \mathbf{v} - {}^{t} (\nabla \mathbf{v}) \right)$$
(7)

Spatial derivatives: The second-order finite differences method Time integration: The fourth-order Runge-Kutta method

# Simulation region and boundary condition



### **Confinement region**

 $0.15L_r \leq r \leq L_r$  ,  $0.5L_r \leq z \leq 2L_r$  $N_r \times N_{\theta} \times N_z = 69 \times 64 \times 121$ 

### Gun region

 $0.175L_r \le r \le 0.65L_r$ ,  $0 \le z < 0.5L_r$  $N_r \times N_{\theta} \times N_z = 39 \times 64 \times 40$ 

### **Boundary condition**

0; All boundaries except for electrode surfaces Non-zero; Electrode surfaces

Bias field is temporally constant.

**Electric field** External electric field  $E_{ini}$ 

0; All boundaries except for the gap between electrodes  $E_r$  of non-zero; Gap between electrodes  $\mathbf{E} = \langle$ 

Heat flux can passes across the all boundaries.

# **Initial condition**



### Numerical axisymmetric MHD equilibrium with finite pressure



**Poloidal flux contour** 

**Radial profile** 

# **Parameters**



Parameters	Normalized quantities	Real quantities
Radius of confinement region $L_r$	1.0	0.5 m
Number density $n_0$	1.0	$5.0 \times 10^{19} / m^3$
Characteristic magnetic field $B_0$	1.0	0.2 T
Initial toroidal current $I_{\rm t}$	1.0	80 kA
Initial temperature T	1.0×10 <sup>-2</sup>	40 eV
External radial electric field $E_{inj}$	1.0×10 <sup>-2</sup>	1.3 kV/m
TF current I <sub>tf</sub>	1.2	96 kA
Initial parallel current density on $axis \lambda_{axis}$	1.6	3.2 /m
Resistivity $\eta$	2.0×10 <sup>-4</sup>	$7.8 \times 10^{-5} \Omega \cdot m$
Viscosity v	1.0×10 <sup>-3</sup>	$2.6 \times 10^{-5}$ kg/m·sec
Conductivity <i>k</i>	1.0×10 <sup>-3</sup>	0.32 W/m·K
Alfven time $\tau_A$	1.0	0.81 µsec

Alfven velocity  $v_{\rm A} \sim 620$  km/s, Magnetic Reynolds number  $S \sim 5000$ 

# Time evolution of external electric field, toroidal current, poloidal flux, and magnetic energy for each toroidal Fourier mode *n*



-- The toroidal current is amplified against the resistive decay during the driven phase.

-- The flux amplification is not observed just after  $t=300\tau_A$  because the plasmoid can not be ejected due to the magnetic pressure of pre-existing ST.

-- The *n*=0 mode is dominant, and configuration is almost axisymmetric.

JCG

# **Polarity of toroidal current and flow**





### Time evolution of poloidal field on poloidal cross section





The plasmoid is ejected from the gun region due to the Lorentz force, and is then merged with the pre-existing ST. As the result, the poloidal flux amplification occurs.





The closed flux surfaces are rebuilt due to the dissipation of the magnetic fluctuation in the COFC.

### Time evolution of flows on poloidal cross section





At  $t=804\tau_A$ , the poloidal flow which moves from the confinement to gun region is induced due to the pressure gradient.

### Time evolution of $\lambda$ profiles on poloidal cross section





The current sheet is caused by approach of the anti-parallel magnetic field lines during the plasmoid ejection.\_

-1.0 -0.6 -0.2 0.2 0.6 1.0

λ



The  $\lambda$  concentrated in the COFC region is diffused to core region, approaching the Taylor state.

### Time evolution of pressure profiles on poloidal cross section





The broad pressure profile is formed in the core region due to the diffusion of pressure from the COFC to core region.

### Time evolution of radial electric field on toroidal cross section



-0.006

-0.018

0.006

0.018



 $E_{inj}=0$  (Decay phase)  $t=818 \tau_{A}$  $t=804 \tau_A$  $t=702 \tau_{A}$  $t=796 \tau_A$  $t=833 \tau_A$  $t=855 \tau_{A}$  $t=877 \tau_A$ Z

The current density is diffused from the COFC to core region due to the positive electric field in the COFC.

### Time evolution of MHD dynamo on poloidal cross section





During plasmoid ejection process, the dynamo effect which diverts power from the gun to core region is observed.



We carried out the nonlinear MHD simulation of the current drive of **high-***q***ST** plasmas by the **M-CHI**.

- 1. During the driven phase, the poloidal flux and the toroidal current are amplified by **the axisymmetric merging of the pre-existing ST plasma with ejected one**.
- 2. During the decay phase, the **ST approaches the axisymmetric MHD** equilibrium state without flow due to the dissipation of magnetic fluctuations to rebuild the closed flux surfaces.
- 3. The positive electric field in the COFC causes the current density or  $\lambda$  to diffuse from the COFC to core region, leading the helicity transport.