



Recent progress in JT-60U experiments  
and  
Plasma diagnostics plan in JT-60SA tokamak

Presented by

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# Outline of JT-60U

**JT-60U**

Fiscal Year	85	90	95	00
$Q_{DT}^{eq}$			0.6	1.05 1.25
Ti (keV)				45 keV
Te (keV)				26 keV

$I_p < 3 \text{ MA}$ ,  $B_T < 4 \text{ T}$   
 $R = 3\text{-}3.5 \text{ m}$ ,  $a = 0.6\text{-}1.0$

P-NBI  
 Perp.: 20MW

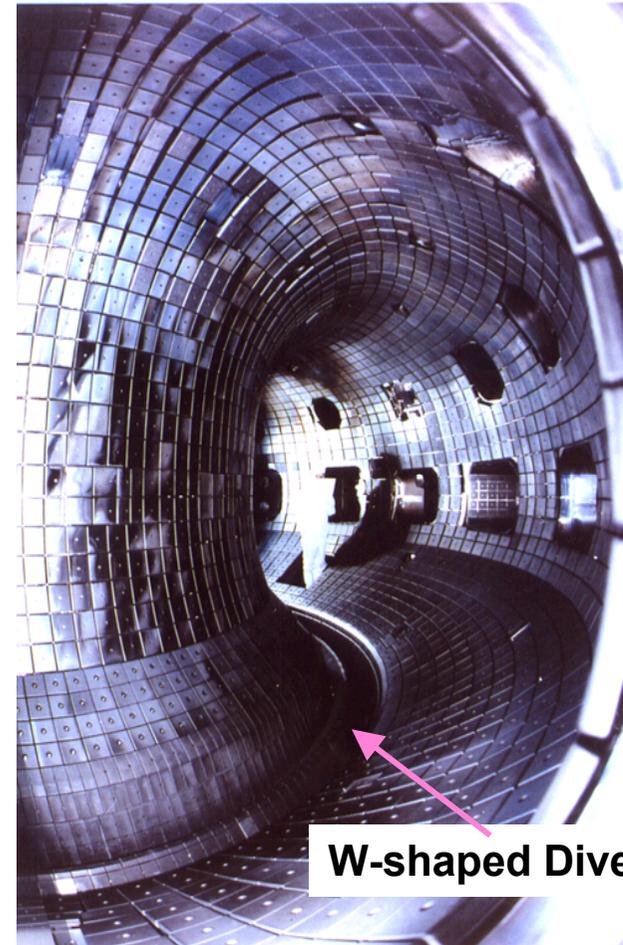
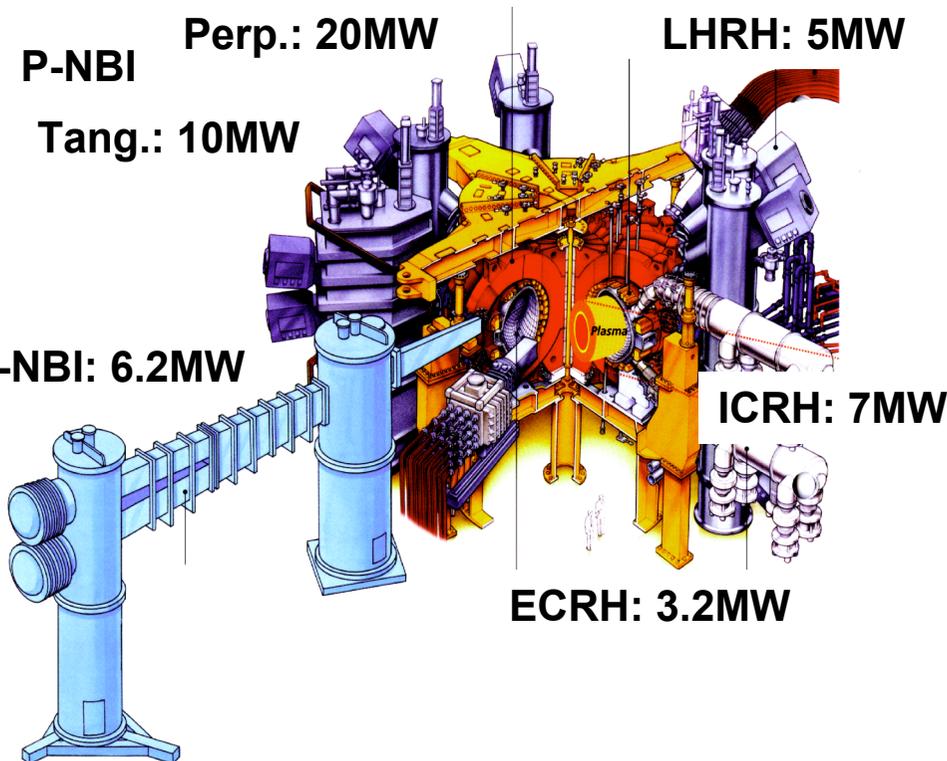
LHRH: 5MW

Tang.: 10MW

N-NBI: 6.2MW

ICRH: 7MW

ECRH: 3.2MW

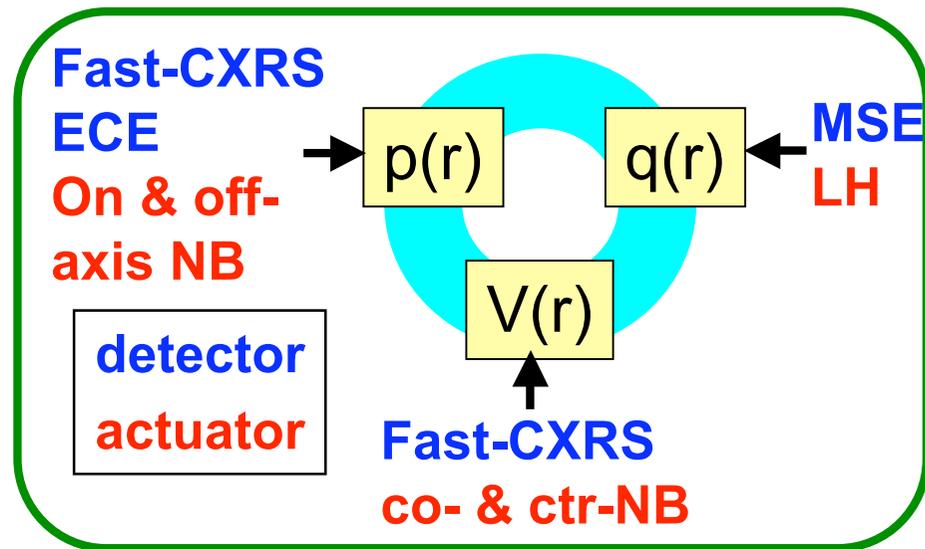
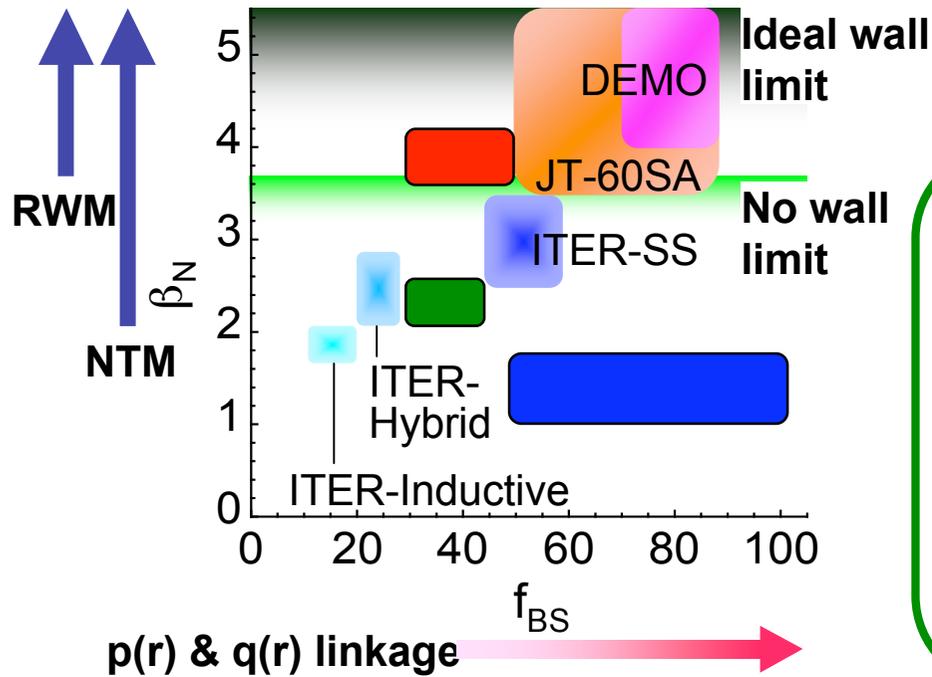


W-shaped Divertor

In-Vessel View

# AT reserch in JT-60 toward DEMO

JT-60U



# Objectives of JT-60 experiments during 2007–2008

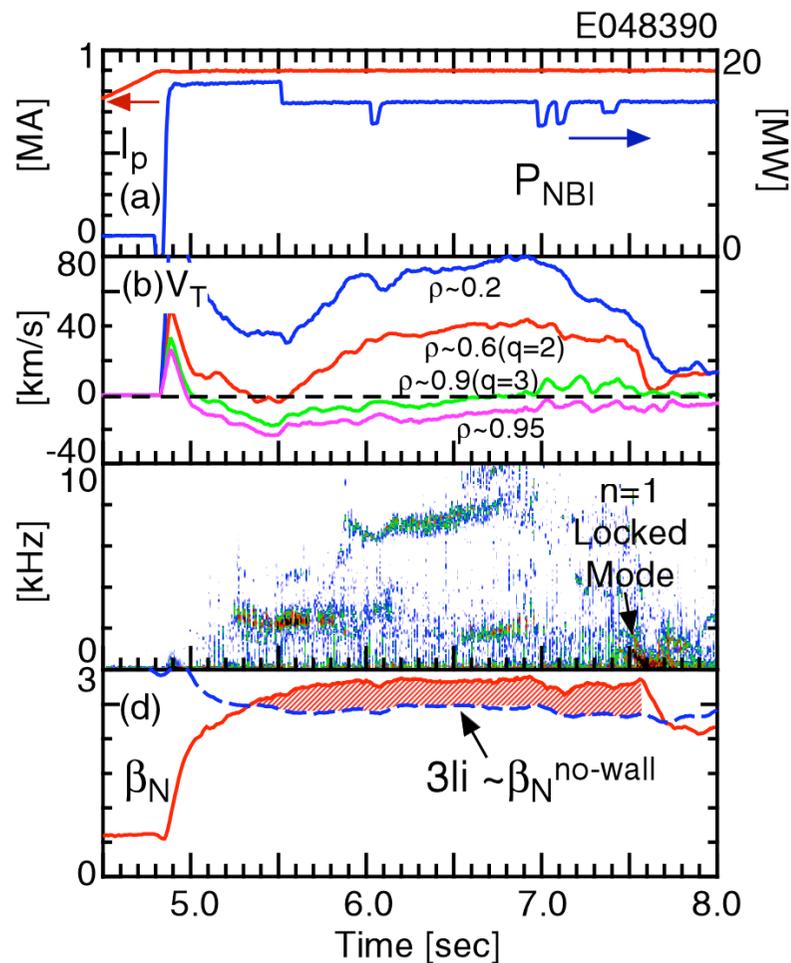
JT-60U

- Extension of operation regime based on steady-state high beta research using modified heating system and optimized control system towards DEMO, ITER and JT-60SA.
- Development of real-time control scheme for burning plasma control and establishment of physics bases for accurate predictions of a plasma performance using newly installed diagnostics.
- Physics R&D for JT-60SA design.
- Targets
  - Achievement of long sustainment of high integrated performance  $\beta_N=2.5-3$ ,  $HH \geq 1$ ,  $f_{BS} \geq 0.4$ ,  $\tau_{\text{duration}} = 25-30 \text{ s}$  ( $\sim \tau_{\text{NB-inj.}}$ )
  - Achievement of long sustainment of high beta exceeding no-wall ideal MHD limit  $\beta_N=3-3.5$  ( $> \beta_N^{\text{no-wall}}$ ),  $\tau_{\text{duration}} = 5-8 \text{ s}$  ( $\sim \tau_R$ )
  - Achievement of long sustainment of high radiation fraction  $f_{\text{rad}} \geq 0.8$ ,  $HH \geq 0.9$ ,  $n_e/n_{\text{GW}} \geq 0.7$ ,  $\tau_{\text{duration}} = 15-25 \text{ s}$  ( $\sim \tau_{\text{wall}}$ )

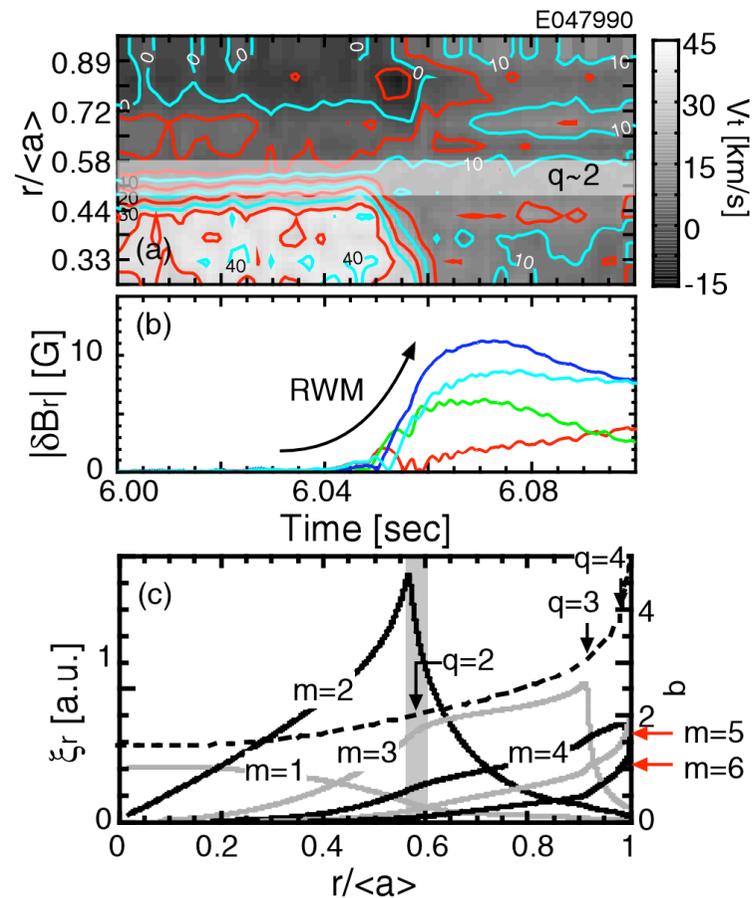
# The dynamics and stability of the resistive wall mode (RWM) are investigated in the JT-60U high- $\beta$ plasmas.

JT-60U

\*  $\beta_N \sim 2.8$  sustained for 2 s above the ideal no-wall  $\beta$ -limit (= 2.4)



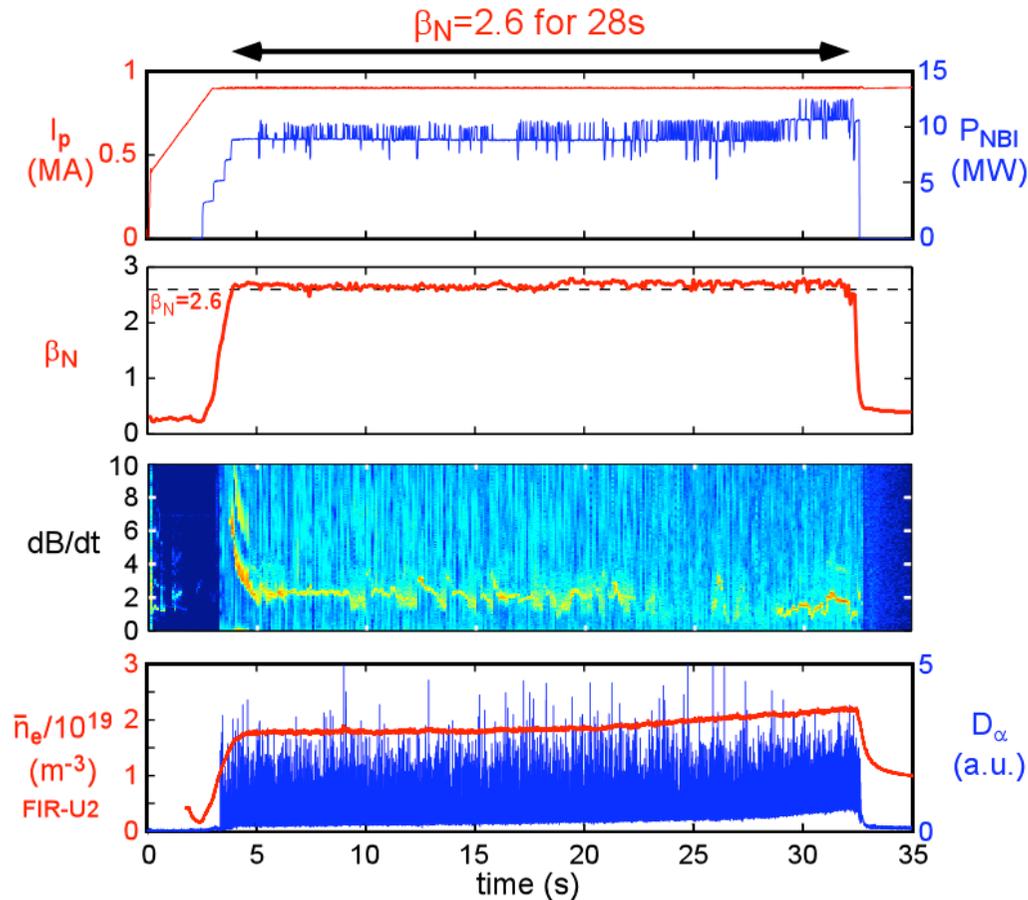
\* Braking of the plasma rotation inside  $q = 2$  surface observed



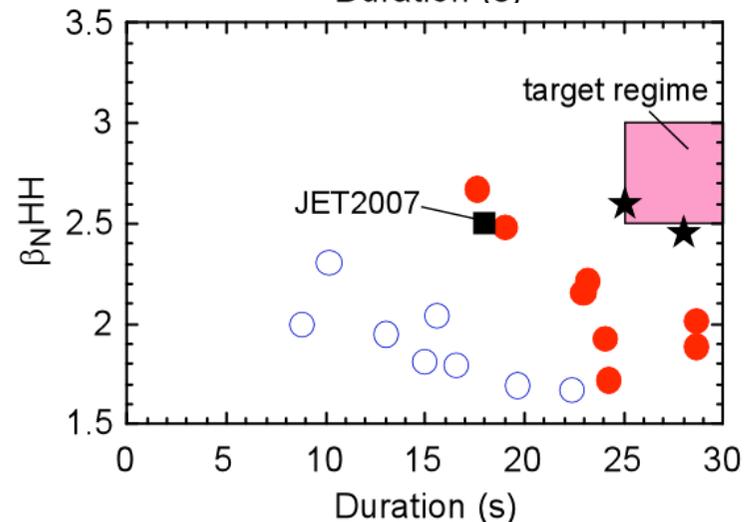
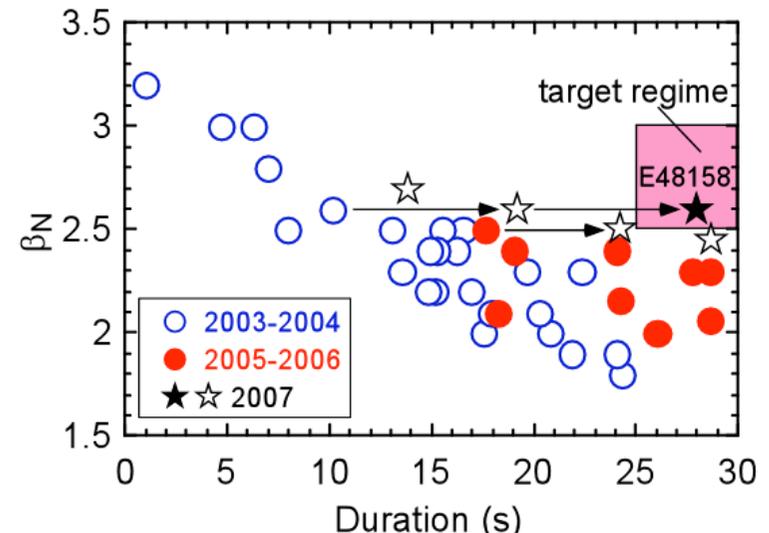
# High integrated performance with long pulse length

**JT-60U**

E48158 was satisfy following condition. “To sustain high integrated performance plasma for 25-30s, whose parameters are  $\beta_N > 2.5$ ,  $H_H \geq 1$  and  $f_{BS} \geq 0.4$ ”

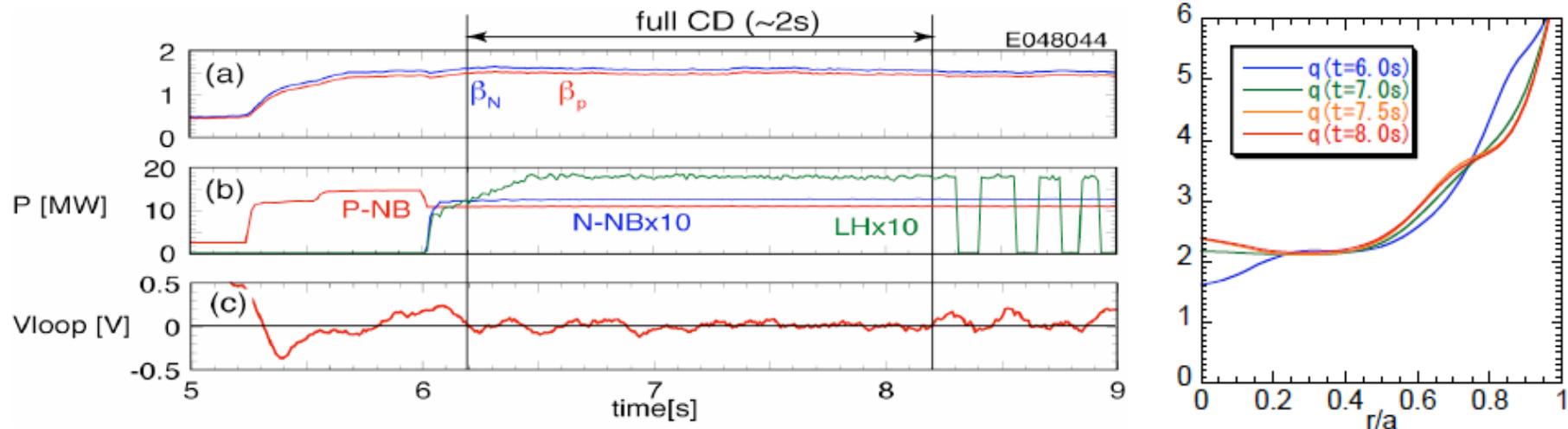


A series of 3 sequential long-pulse discharges and 7 hours GDC were effective to reduce recycling.



# Fully non-inductive having relaxed current profile with high $f_{BS}$ ( $= 0.5$ )

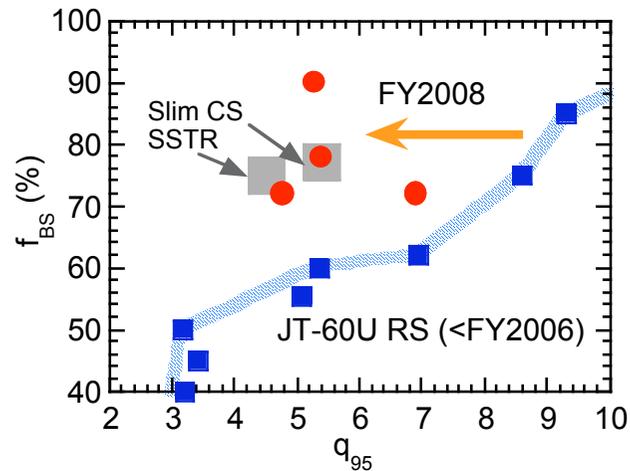
JT-60U



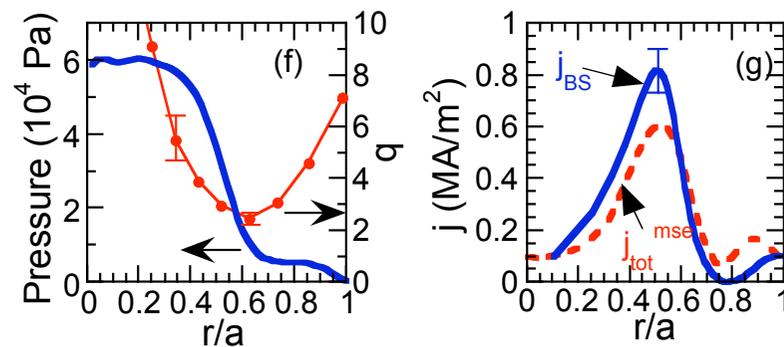
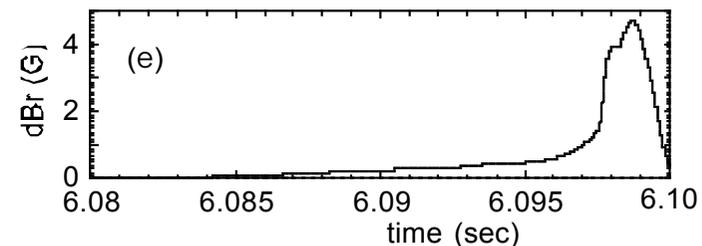
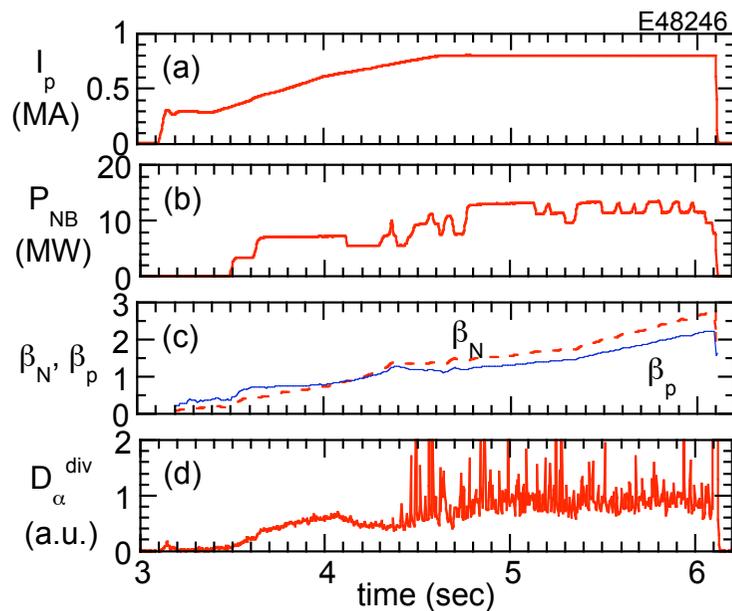
- Full CD condition was obtained in the high  $\beta_p$  plasma, assisted by N-NB (320keV,  $\sim 1.2$ MW) and LHCD ( $\sim 1.8$ MW) in E48044 ( $I_p=0.8$ MA,  $B_t(0)=2.3$ T,  $q_{95}=5.8$ ).
- Current drive fraction (ACCOM code)
  - Bootstrap current ; 50%
  - LHCD; 26%
  - N-NB CD ;24%
- Weak shear regime at  $q_{min}=2.1(>2)$  and  $q(0)=2.4$ .
- No MHD activity was observed (1/1, 3/2, 2/1 mode-free q profile).

# Reversed shear plasmas with high $f_{BS}$ were extended to the low $q_{95}$ ( $\sim 5$ ) and high $\beta_N$ regime

**JT-60U**



- By utilizing large volume configuration close to the conductive wall for wall stabilization ( $d/a \sim 1.2$ ), beta limit is significantly improved.
- As a result,  $\beta_N \sim 2.7$  and  $\beta_p$  was achieved in RS plasma with  $q_{min} \sim 2.4$ .



# Tungsten accumulation found to be a strong function of the plasma rotation speed.

**JT-60U**

There are strong arguments on the choice of Tungsten as the first wall materials in ITER.

## Merit (compared with CFC)

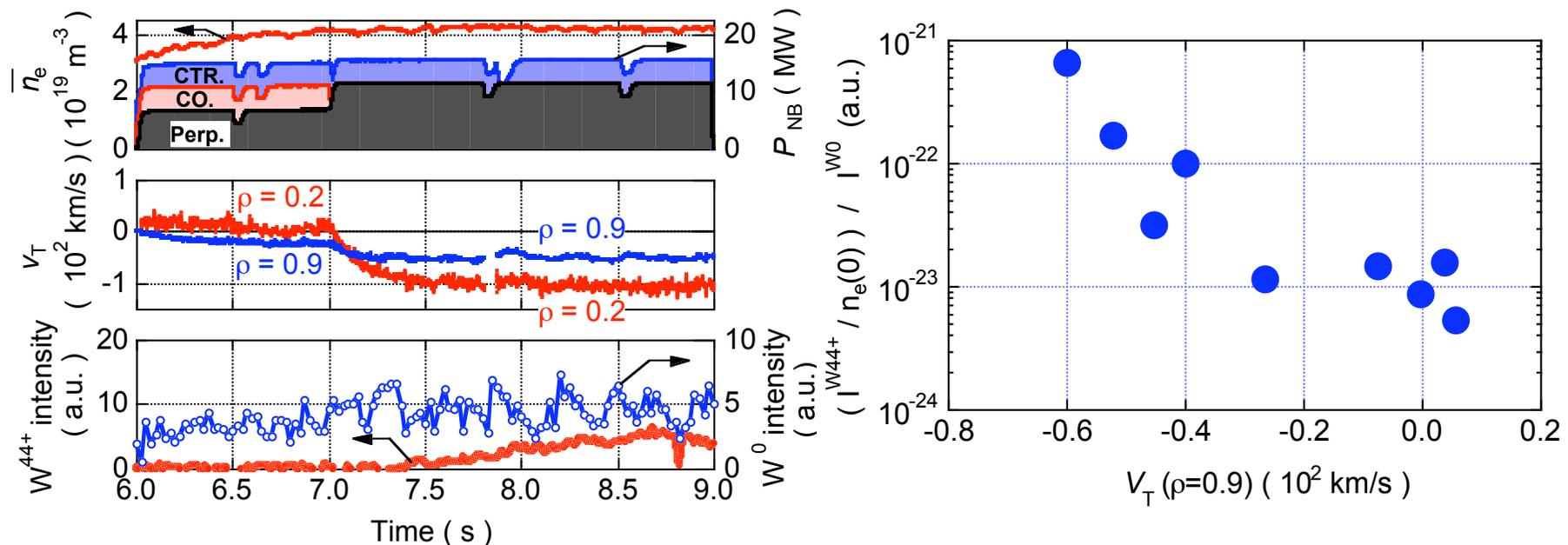
- Small amount of tritium retention.
- Small sputtering yield.

## Demerit

- Strong radiator in high Te plasmas.
- Small amount of W allowed.

Plasma rotation changed when the outer strike point is on the W coated tiles. (~5 % of toroidal coverage).

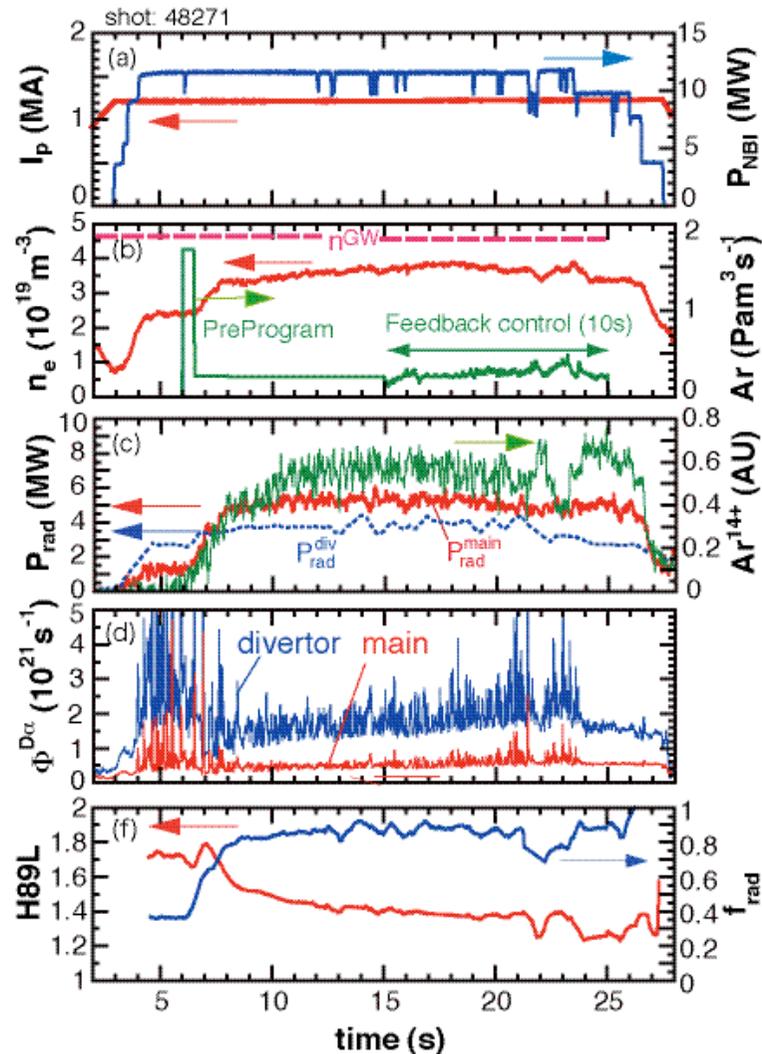
Tungsten accumulation rapidly increases with VT in counter Ip direction.



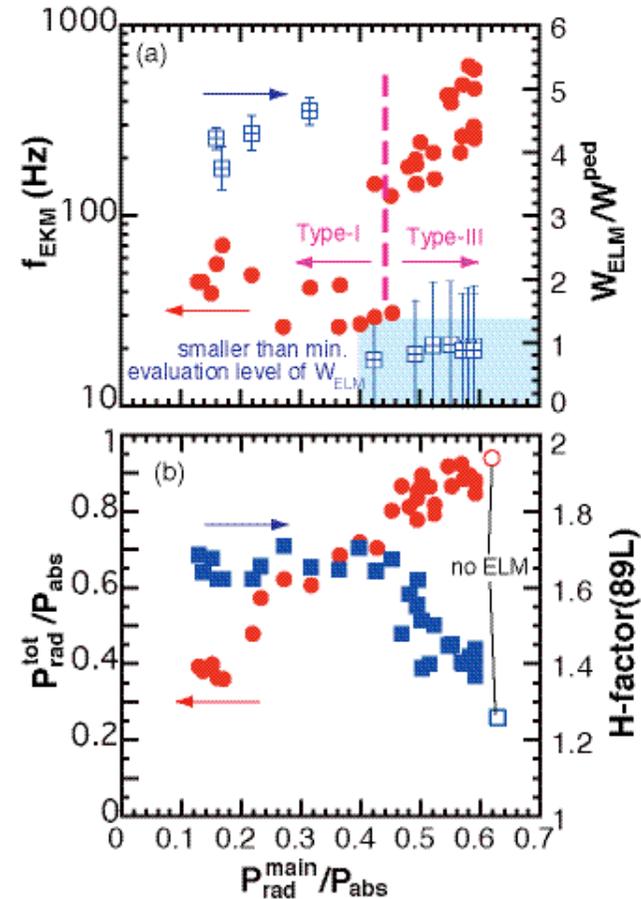
# Long-sustainment of good energy confinement plasma with low ELM energy loss due to large radiation fraction by Ar seeding.

JT-60U

- Large frad ( $= 0.82-0.9$ ), high  $n_e/n_{GW} = 0.74-0.8$  and relatively good  $H_{89L} (= 1.37-1.45)$  was sustained for 16s.

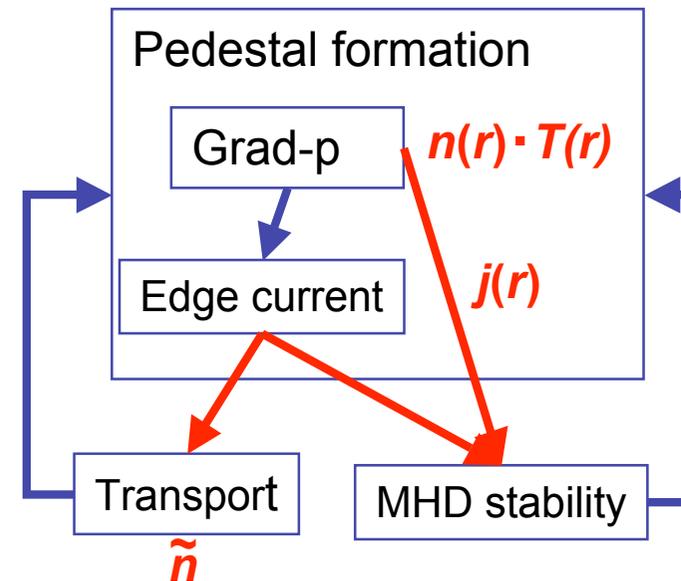
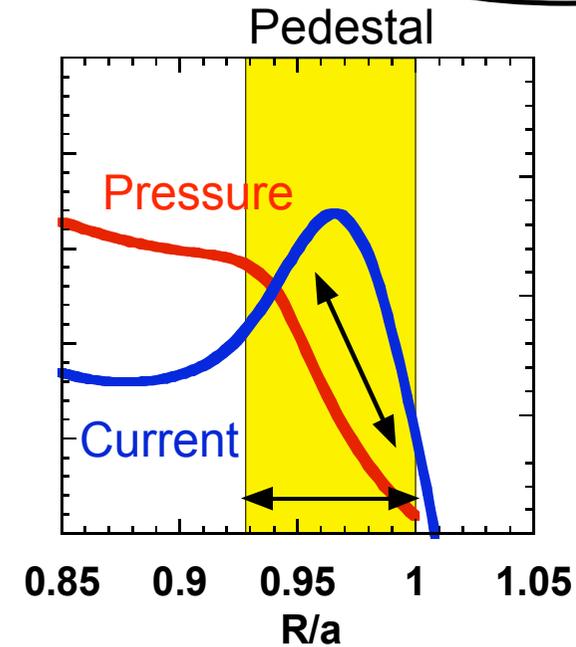
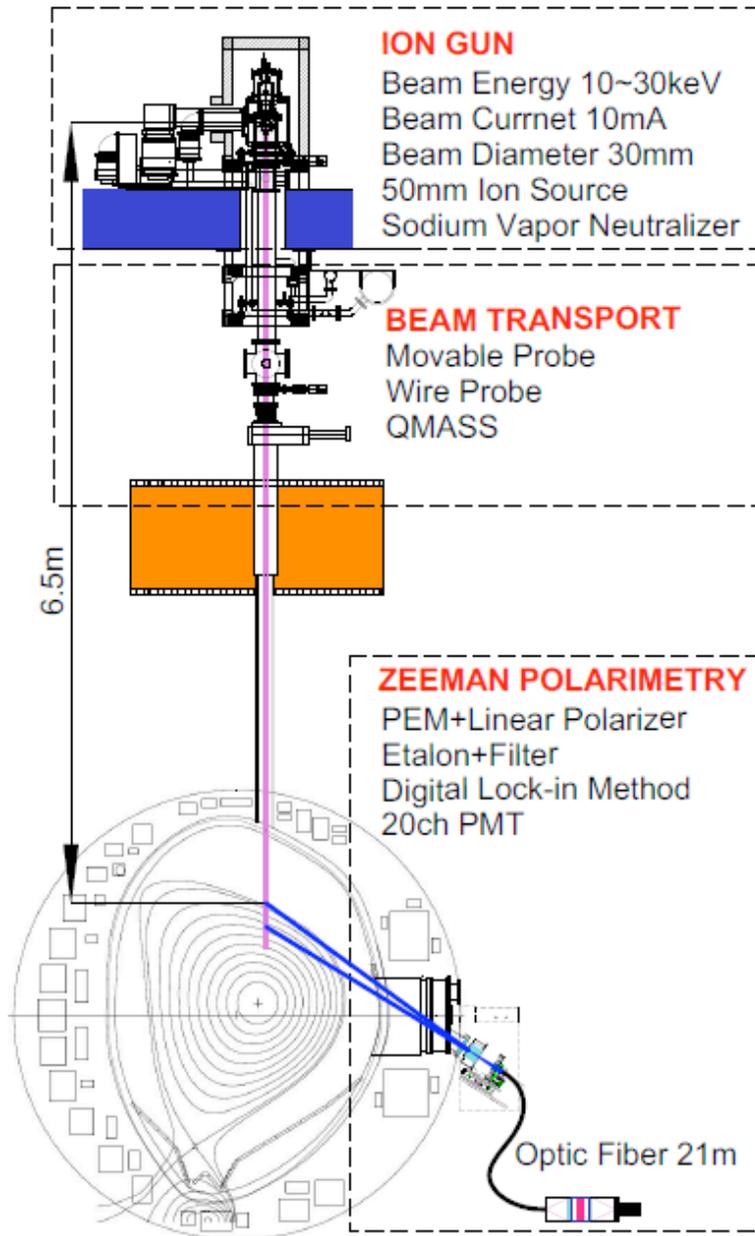


Low ELM energy loss  
( $W_{ELM}/W_{ped} < 1.7\%$ )



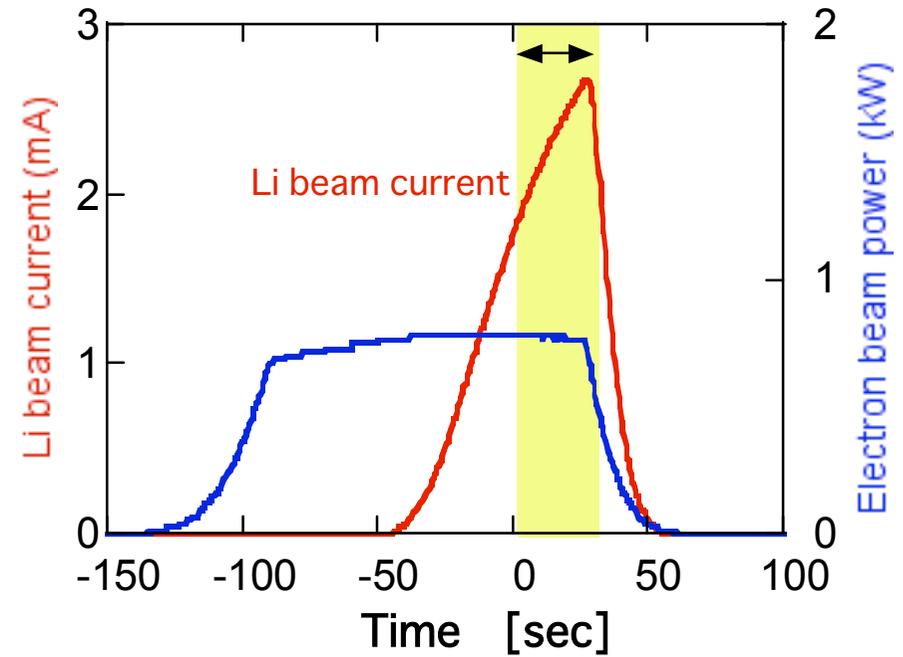
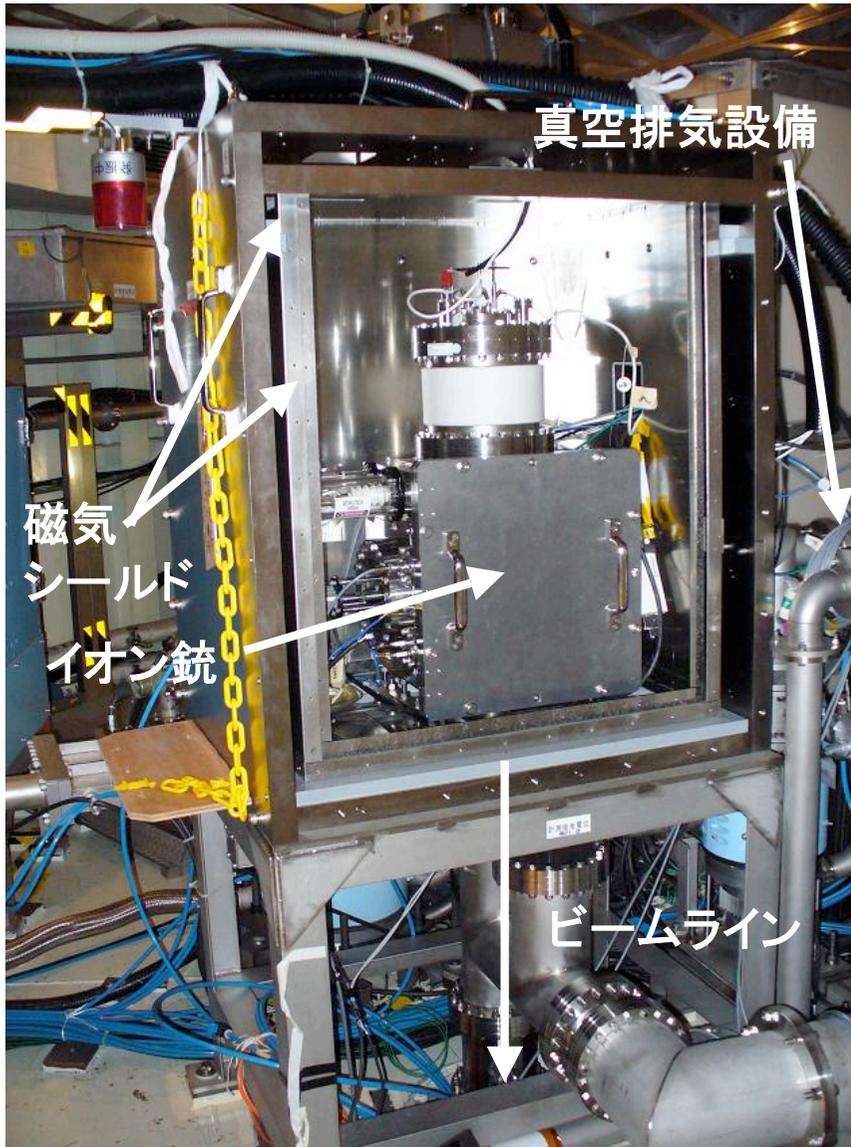
# Lithium Beam Probe is being developed for JT-60U

JT-60U



# Li beam gun installed in JT-60

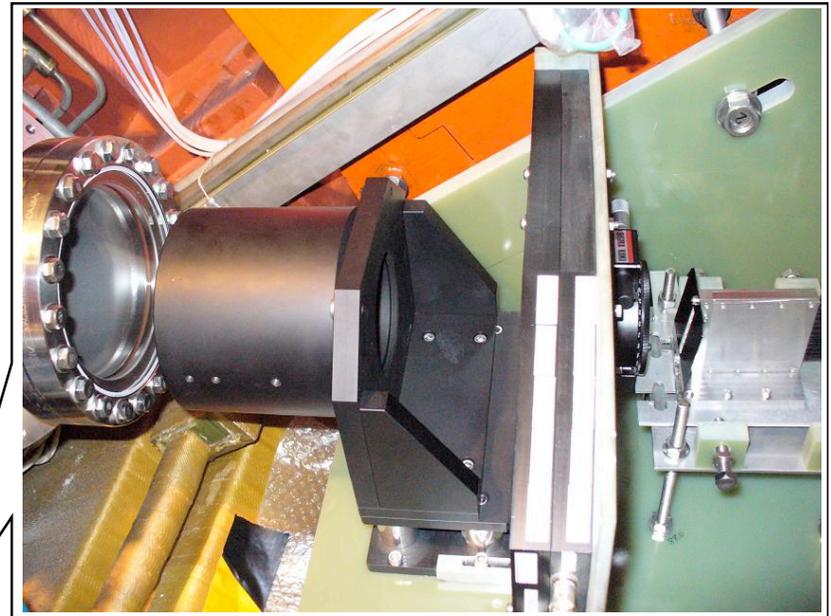
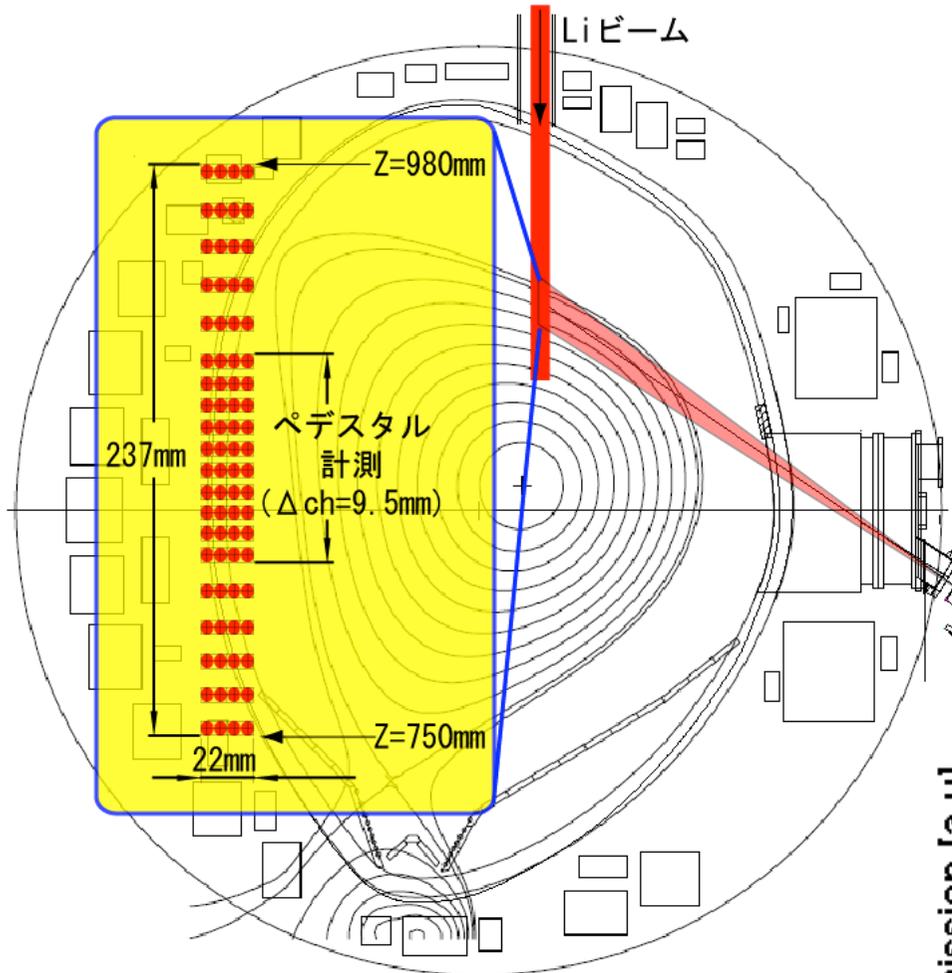
JT-60U



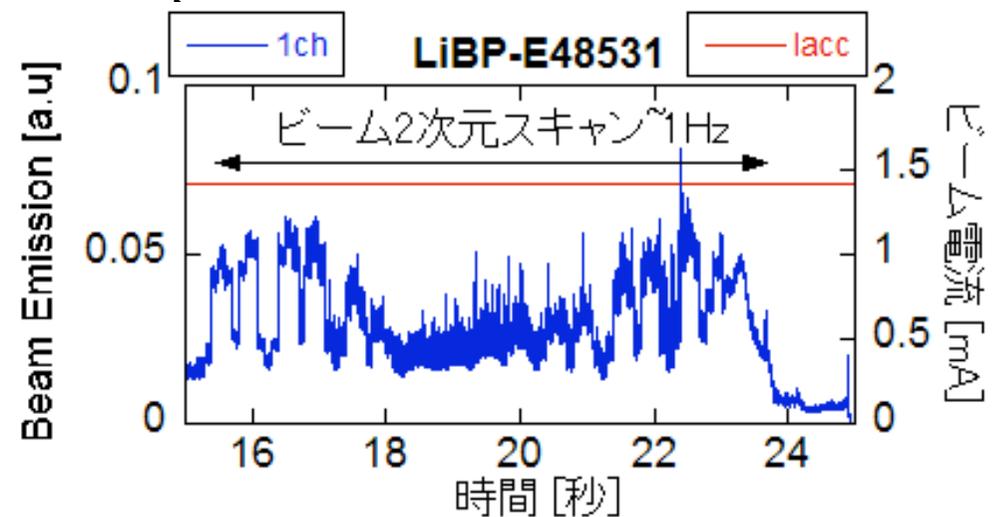
■ Magnetic shield 300G → 0.1G

# Li BP is now ready for the plasma density measurement

JT-60U

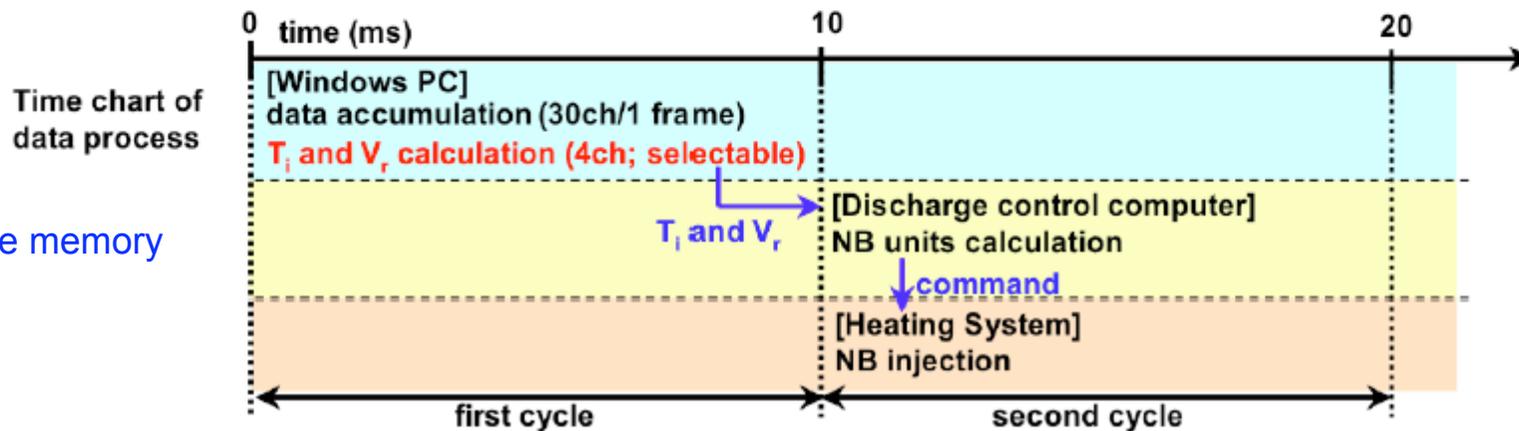
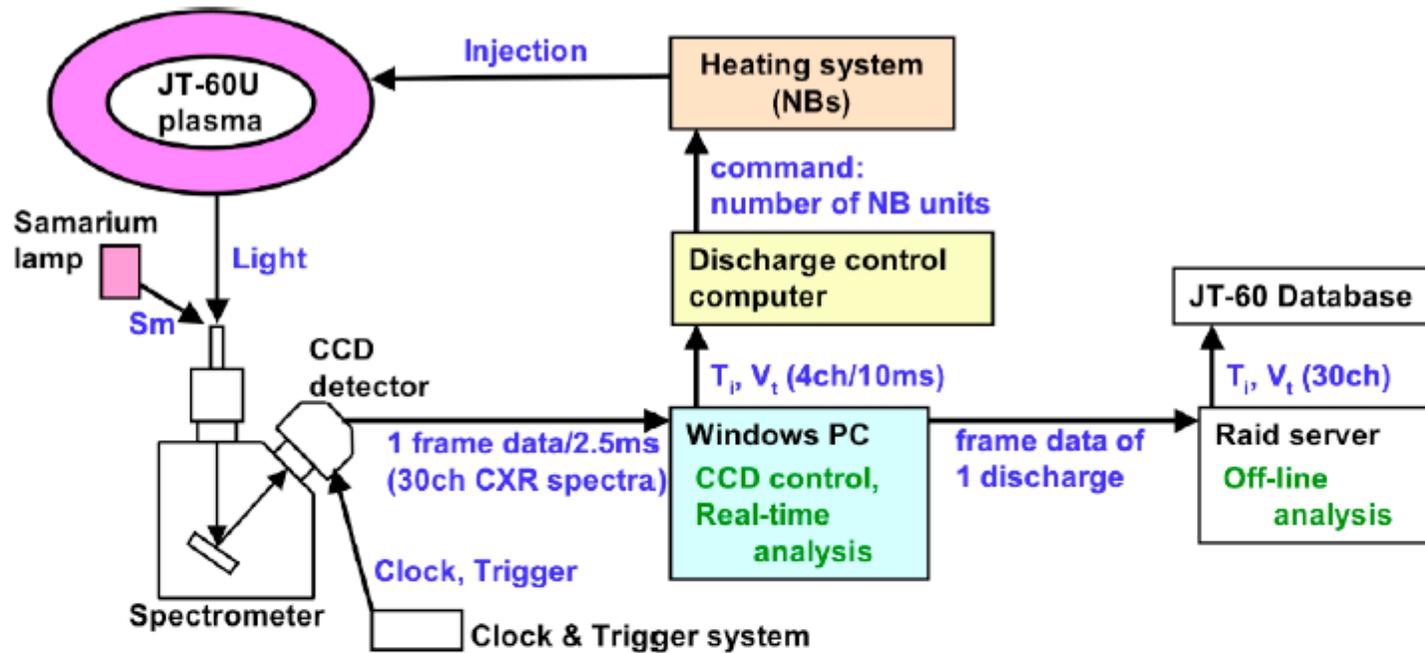


■ Data Acquisition  
20ch、1MHz sampling



# Real time control using fast CXRS

JT-60U

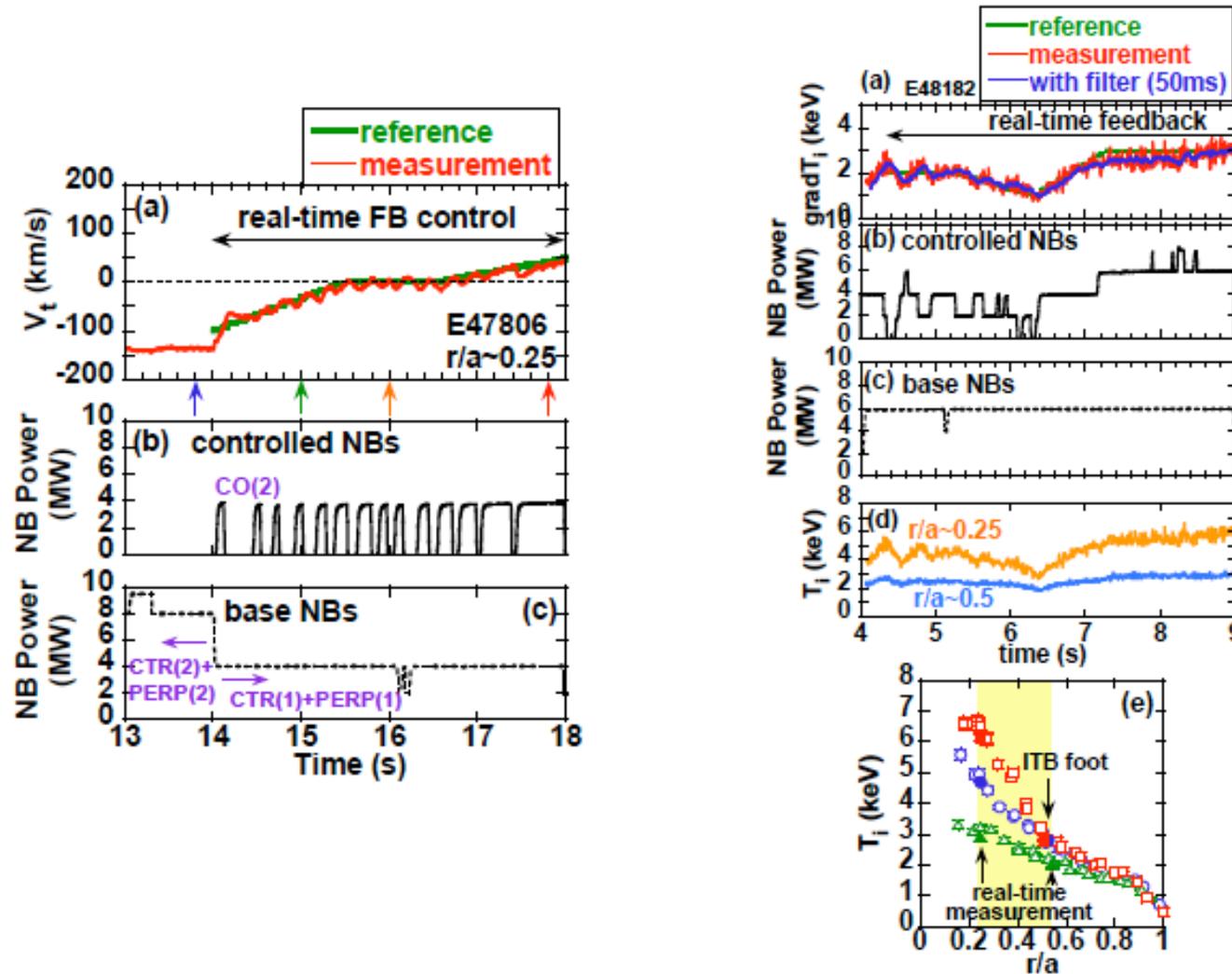


Reflective memory

# $V_\phi$ and grad $p$ control in JT-60U

JT-60U

- M. Yoshida et al, submitted to Fusion Engineering and Design (2008)





# Simultaneous plasma control in JT-60SA

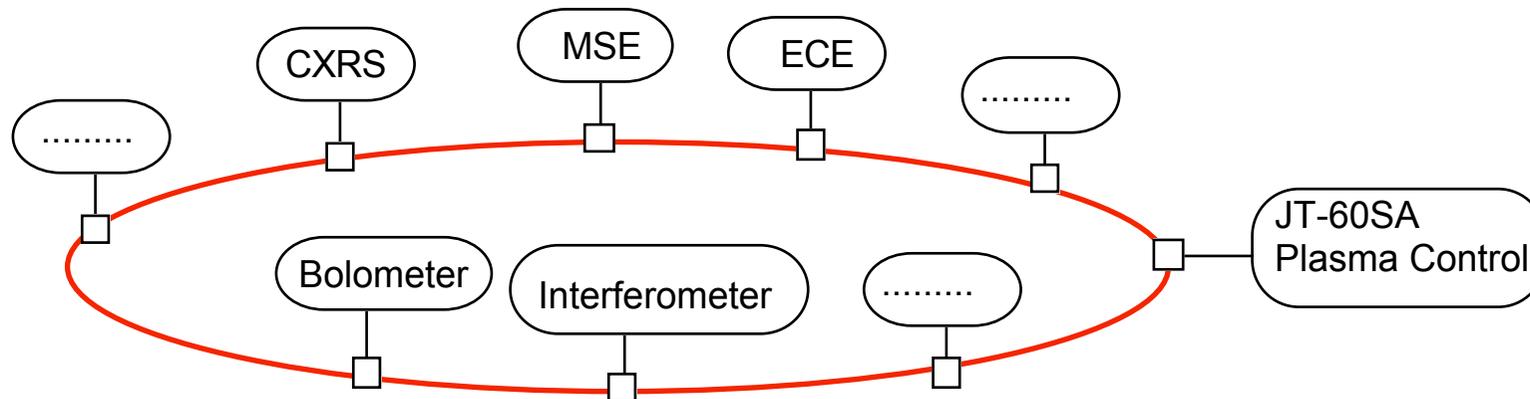
JT-60SA

*The current density profile  $j(r)$  is strongly linked with the plasma pressure profile  $p(r)$  in high  $f_{BS}$  plasmas.*

*In order to achieve high  $\beta_N$  plasmas above the no wall limit in steady state, the NTM modes and RMW must be suppressed by the real time control of the plasma profiles, such  $j(r)$ ,  $p(r)$  and  $V_\phi$ , simultaneously.*

*In addition to the main plasma parameters, the detached divertor must be sustained and controlled in the divertor plasma.*

- Reflective memory loop for real time control



Each diagnostic system provides the processed plasma parameters in real time to this loop, which is refreshed in time scale of the plasma control.



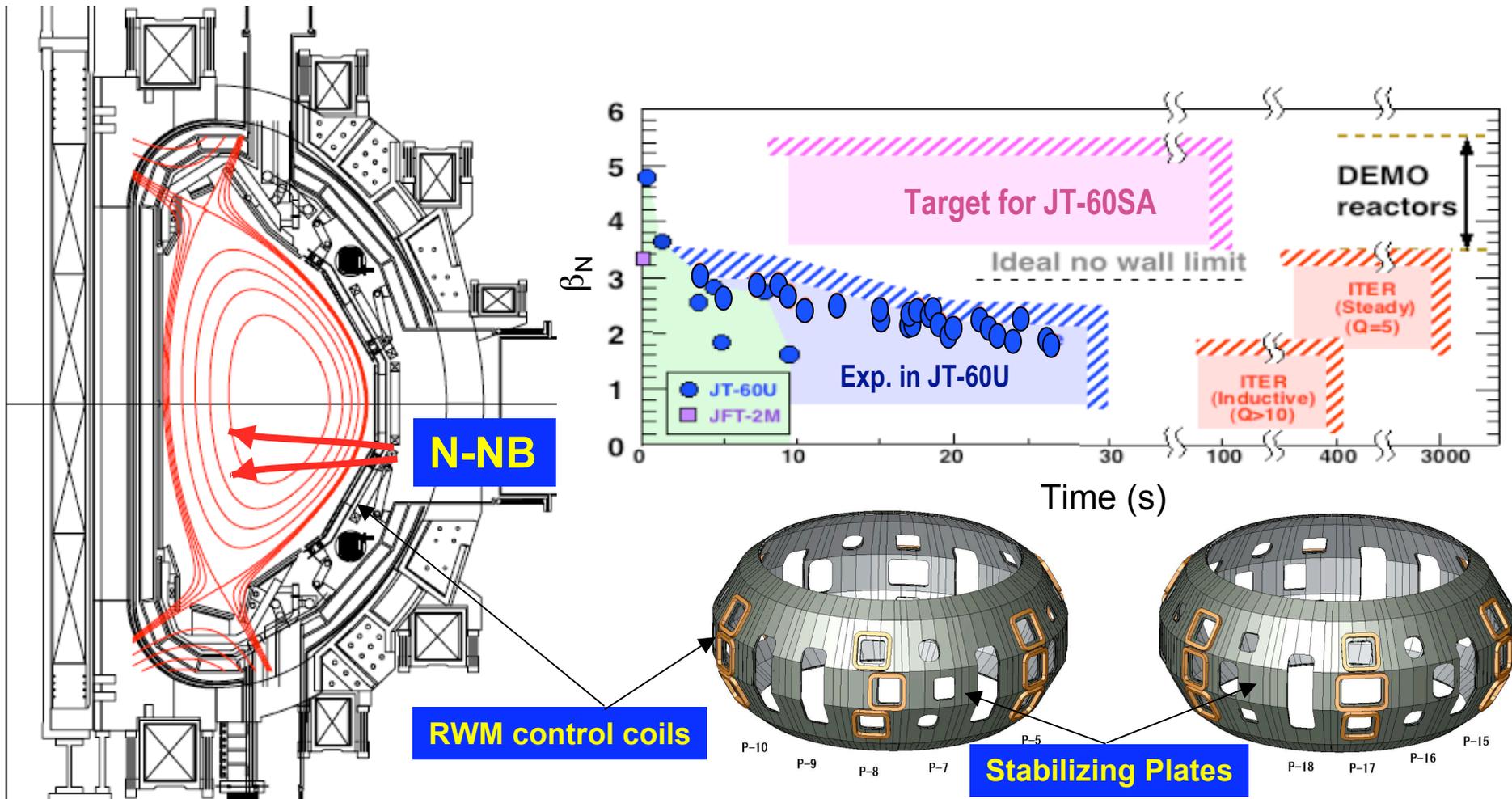
# Plasma diagnostics plan in JT-60SA tokamak



# Mission of JT-60SA is to support and supplement ITER toward DEMO.

JT-60SA

JT-60 SA is a combined program of ITER Satellite Tokamak Program of JA-EU and Japanese National Program





# Sharing of construction of JT-60SA

JT-60SA

## Superconducting Magnets

- Poloidal field coils (CS & EF) 
- Toroidal field coils 

## Cryostat



## Vacuum Vessel



## NBI system



## Diagnostics



## Remote Handling System



## Cryogenic System



## Power Supply



## ECH System



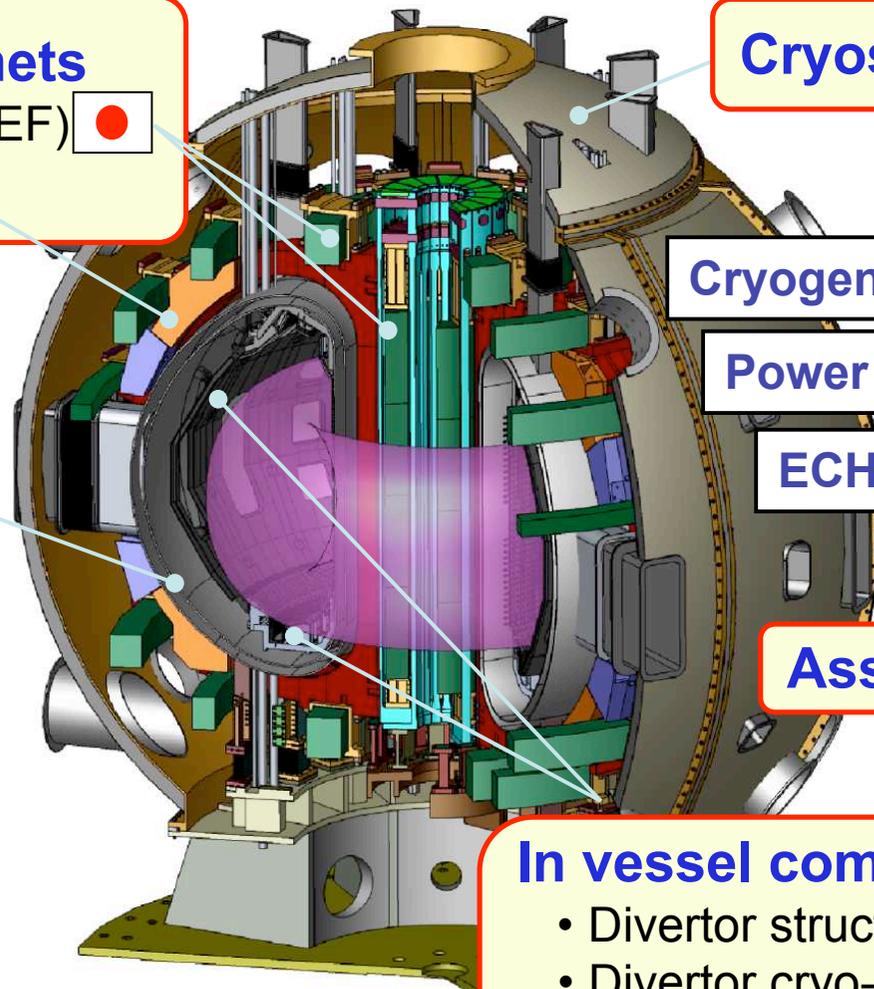
## Assembly



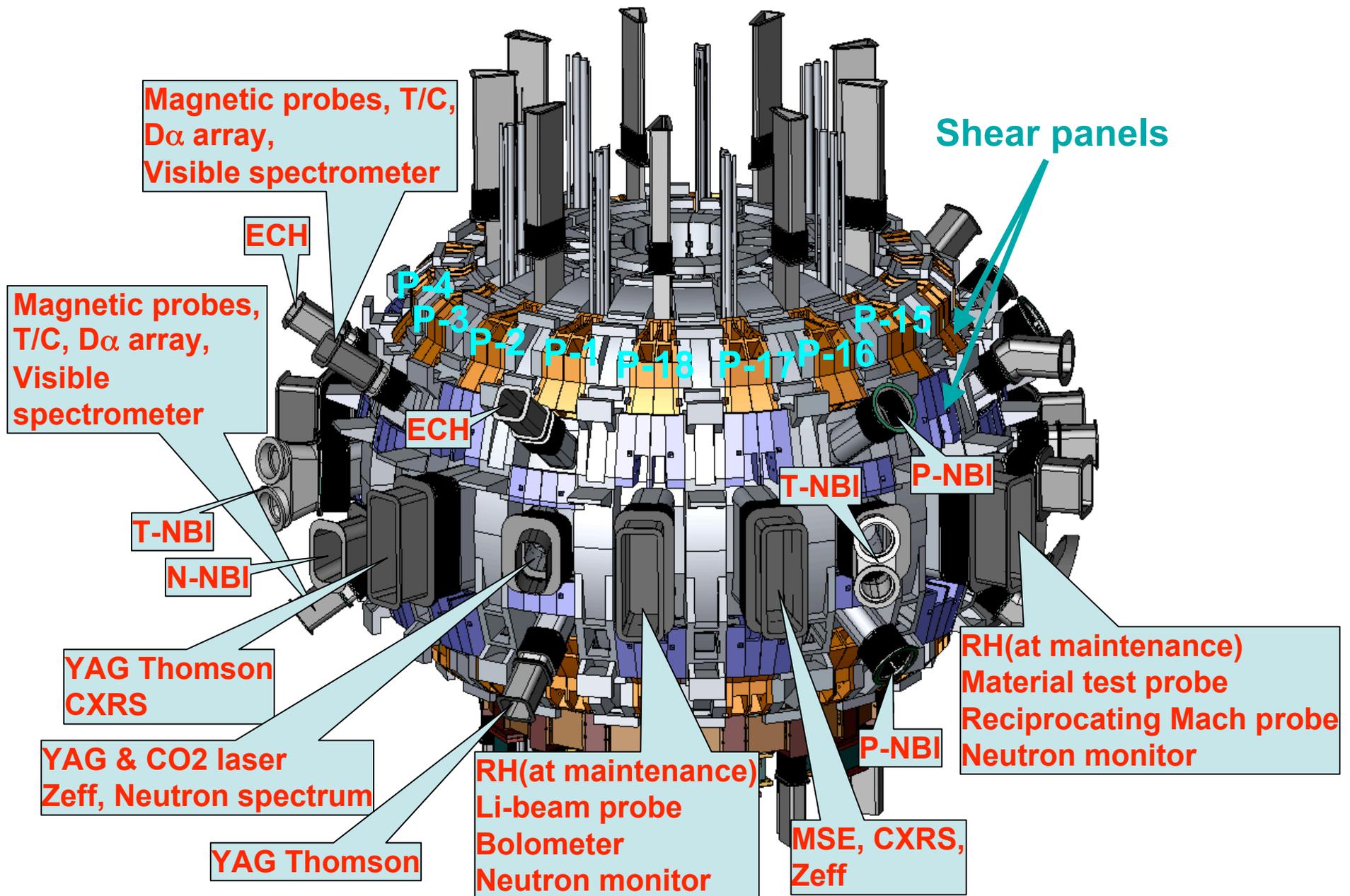
## In vessel components



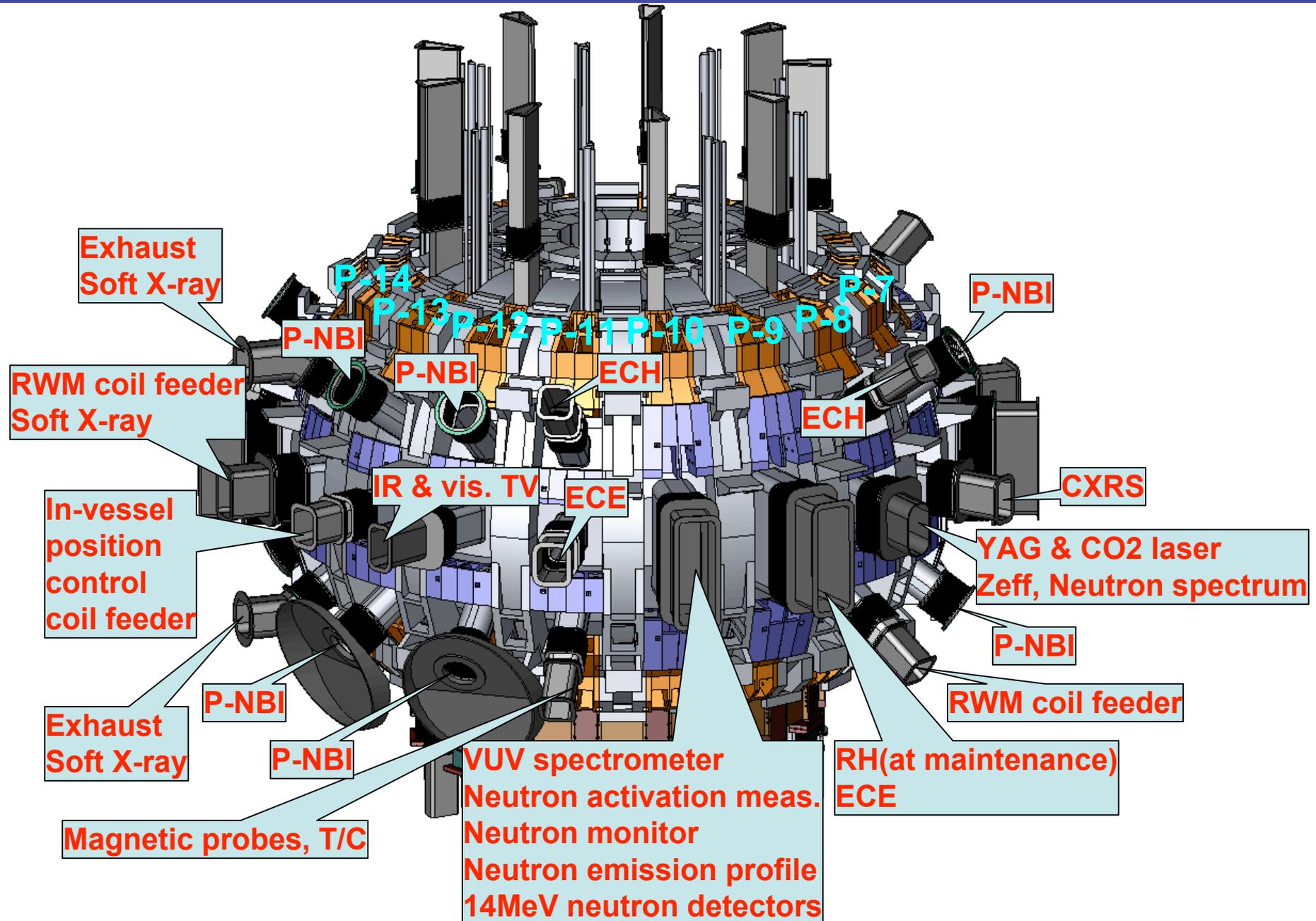
- Divertor structures
- Divertor cryo-pumps
- First walls
- In vessel coils



# Outline of Port Allocation(P-4, P-3, ---, P-16, P-15)



# Outline of Port Allocation(P-14, P-13, ---, P-8, P-7)





# Specification of the diagnostic systems (SA-CDR Appendix)

JT-60SA

## • For Machine Protection and Operation

System	Specification
Neutron monitor	Total DD neutron emission rate measurement. Sets of $^{235}\text{U}$ and $^{238}\text{U}$ fission chambers will be installed at two or three toroidal positions and one micro fission chamber ( $^{235}\text{U}$ ) will be installed inside a horizontal ports.
Neutron activation measurement	Calibration measurement for the neutron monitor. The main components of the present system in JT-60U are reused. (Ge detector for gamma ray).
Visible TV	Plasma and First wall are monitored with visible light images. This system consists of the endoscope, visible camera and image processing hardware.
D $\alpha$ emission monitor	Particle recycling measurement. The main components of the present system in JT-60U are reused except for the objective optics. 32 ch (8 ch for the lower divertor, 8ch for the upper divertor, 8ch for the lower baffle and 8ch for the
Divertor Langmuir probe	Divertor plasma parameters ( $T_e$ , $n_e$ , potential etc.) measurement. Two sets of ~ 24ch arrays are expected both for the lower and upper divertor plates, including dome. Additional channels to cover the baffles and first walls are planned.
Infrared TV camera (divertor)	Profiles of temperature and heat flux density on the divertor are measured. IR images of targets in the lower and upper divertor are transferred via IR endoscopes and then monitored by IR cameras.

## • For Fundamental Parameter Measurement

System	Specification
YAG laser Thomson scattering system	Te and ne profile measurement. This new system consists of the tangential injection of YAG laser beam and three detection optics (edge, core and inside). The repetition frequency of YAG laser is 50Hz. The inside and edge optics are deferred to the operational phase.
CO2 laser interferometer / polarimeter (tangential)	Line density (tangential chord) measurement. The specifications will be similar to those of the present system (e.g., resolution with 1 ms sampling is $\sim 0.5 \times 10^{19} \text{ m}^{-2}$ , the resolution with 5ms sampling is $\sim 2 \times 10^{19} \text{ m}^{-2}$ ).
CO2 laser interferometer / polarimeter (vertical) *	Line density (vertical chord) measurement. The specifications will be similar to those of the tangential system. This system is deferred to the operational phase.
ECE diagnostics (Fourier transform spectrometer, Grating polychromator, Heterodyne radiometer)	Te profile measurement. Time resolutions are as follows, FTS: 25ms & typically $\sim 20$ ch depending upon Bt & configuration, GP: 20 $\mu\text{s}$ & 40ch, HR: 20 $\mu\text{s}$ & 24ch (+24ch capability), respectively.
Charge exchange recombination spectroscopy	Ti and the plasma rotation profile measurement. This system consists of the toroidal system ( $\sim 18$ ch) and the poloidal system for the core plasma ( $\sim 18$ ch) and the poloidal system for the edge plasma ( $\sim 8$ ch). The spatial resolution of 3 ~ 6 cm and the time resolution of up to 2.5 ms are expected.
Z <sub>eff</sub> monitor (visible bremsstrahlung emission)	Z <sub>eff</sub> profile measurement. Number of the tangentially viewing chords is 14 ch, time resolution 1 ms.
VUV spectrometer	Spectrum measurement of VUV light from the main plasmas. Measurable wavelength is from 0.5 to 40 nm with 20ms time resolution.
Motional Stark effect polarimeter	Plasma current density profile measurement. Spatial resolution $\sim 10$ cm. Time resolution $\sim 10$ ms.
Bolometer (main, divertor)	Plasma radiation profile measurement. In addition to the resistive bolometer arrays, the imaging video bolometer (IRVB) systems, which is underdeveloped

\* Deferred to the operational phase.

## • For Physics Understanding

System	Specification
Soft X-ray detector array	MHD fluctuation measurement. Four detector arrays provide 80 chords, which are used to reconstruct emissivity profiles of soft X-ray from the main plasmas.
Neutron emission profile monitor *	Measurement of DD neutron emission profile in the core plasma. The main components of the present system in JT-60U are reused.
14 MeV neutron detectors *	DT neutron measurement. The main components of the present system in JT-60U are reused. This system is deferred to the operational phase.
Neutron spectrometers *	Energy spectrum of DD neutron measurement. The main components of the present system in JT-60U (time resolution ~0.1s) are reused.
Infrared TV camera (first wall) *	First wall are monitored with IR light images. This system consists of IR endoscopes, IR cameras and image processing hardware.
Li-beam probe *	Measurement of current density and $n_e$ in the edge plasma. The system consists of the Li-Beam injector and the detection optics. This system is underdeveloped in JT-60U and reused in JT-60SA.
Reflectometer *	Plasma density fluctuation measurement. The main components of the present system in JT-60U are reused. O mode, 3ch (34GHz, 36 GHz, 48GHz)
Reciprocating Mach probe *	SOL plasma parameters ( $T_e$ , $n_e$ , potential etc.) measurement. The basic design is similar to the present system in JT-60U are reused. This system is delayed to the operational phase.
Visible spectrometer for the divertor	Multi-channel spectrometer of visible light from the divertor plasmas. Low wavelength resolution spectrometer (instrumental width 0.05nm, 16ch), High wavelength-resolution spectrometer (instrumental width 0.006 nm, 16ch) and Survey spectrometer (wavelength band 500 nm, 100ch) are equipped.
VUV spectrometer for the divertor *	Spectrum measurement of VUV light from the divertor plasmas. The main components of the present system in JT-60U are reused. Measurable wavelength is from 2.5 to 130 nm with 20ms time resolution.
Neutral gas pressure gauge (Penning gauge, Fast response ionization gauge)	Several sets of penning gauges (for pumping regions and main chamber region) and fast response ionization gauges (for the upper and the lower divertor regions).

\* Deferred to the operational phase.

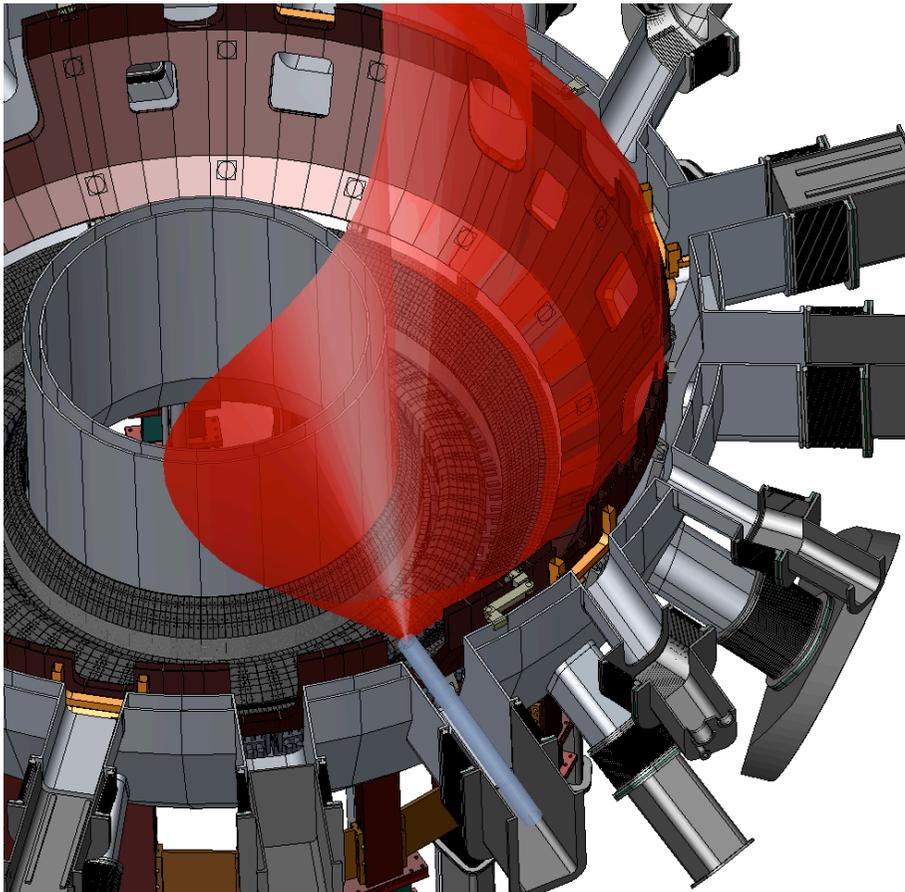


# Wide angle IR/Visible periscope

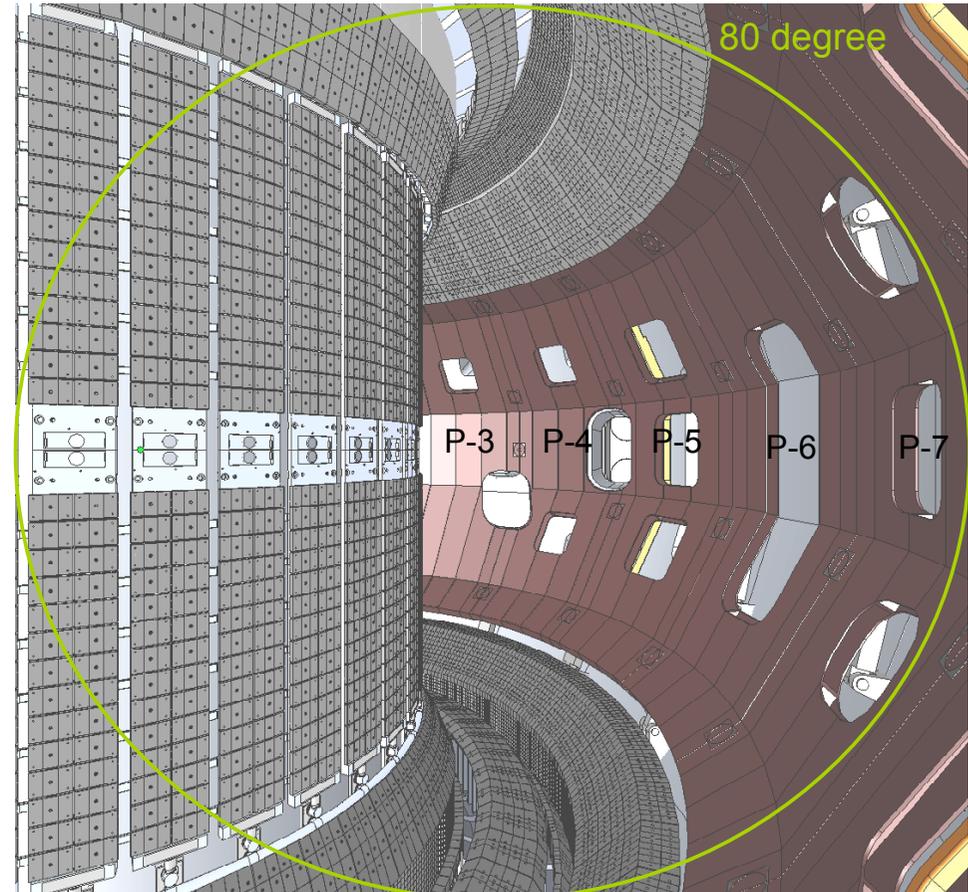
## Visible TV camera ( Plasma and First wall)

JT-60SA

Bird eye view



Inside vessel

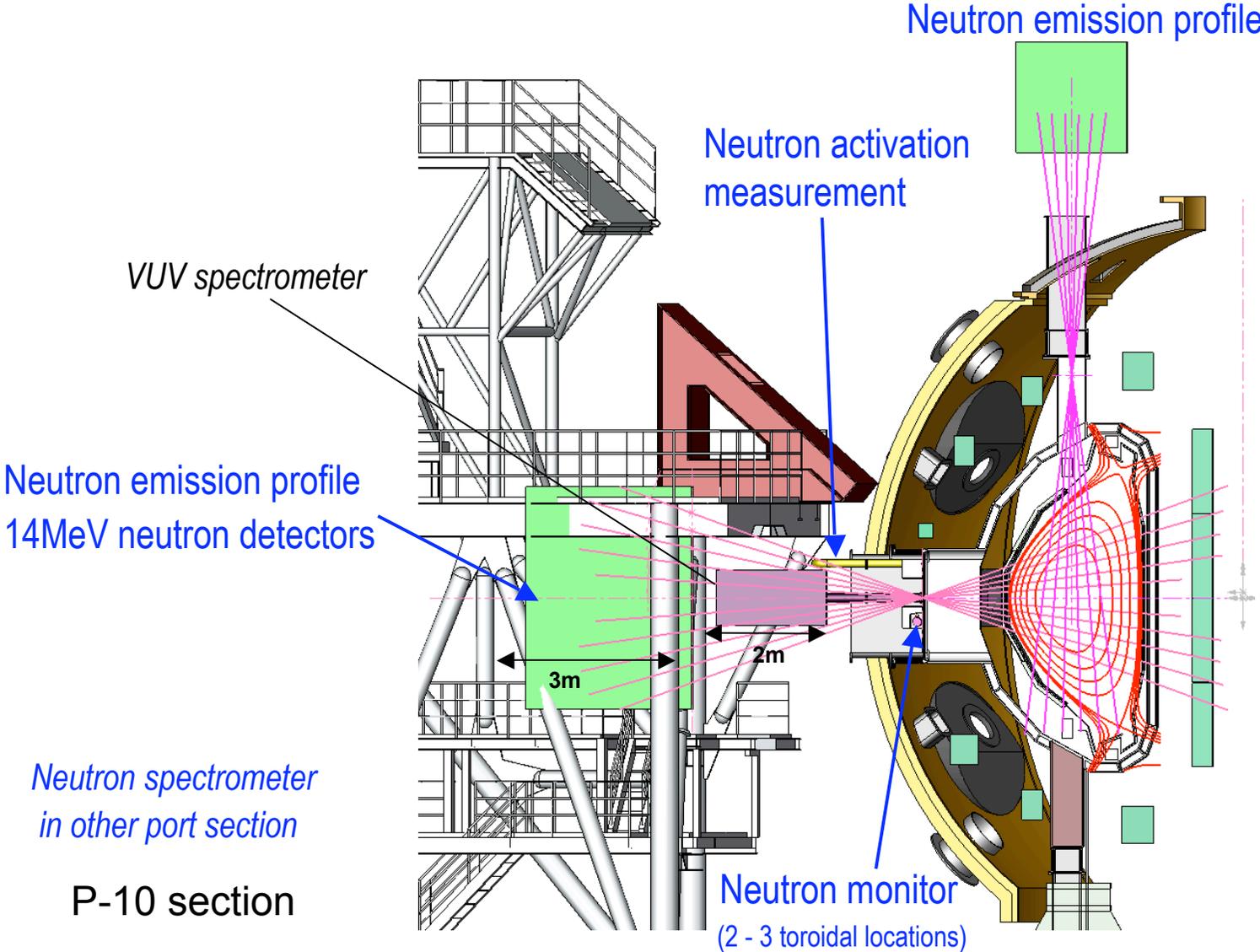


# Neutron measurements



Neutron monitor, Neutron activation measurement  
Neutron emission profile monitor, 14MeV neutron detectors, Neutron spectrometer

JT-60SA

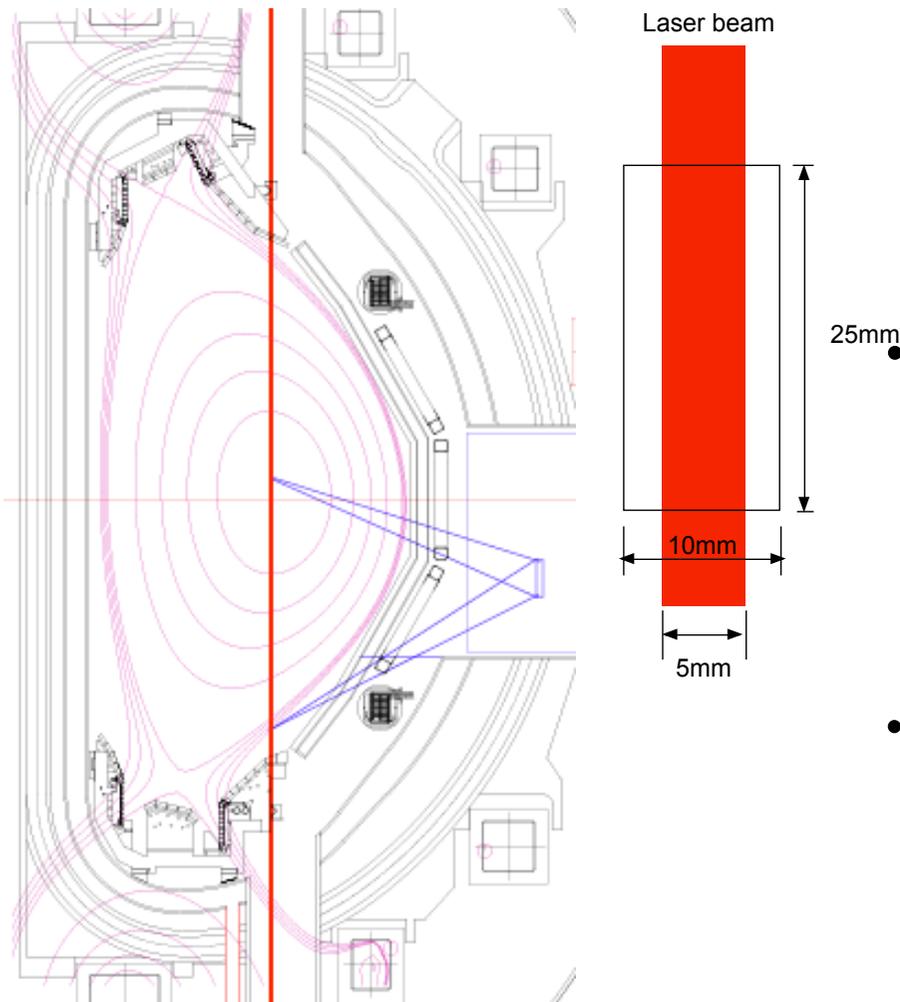




Enhancement of the spatial resolution from JT-60U and capabilities to measure  $T_e = 30$  keV is required.

## YAG laser Thomson scattering

JT-60SA



*Good spatial resolution is required for  $n_e$ ,  $T_e$  and  $T_i$  measurement on the ITB ( $\Delta/a \sim 0.1$ ) in the strong reversed shear plasmas and on the edge transport barriers ( $\Delta/a \sim 0.05$ ) in H-mode plasmas.*

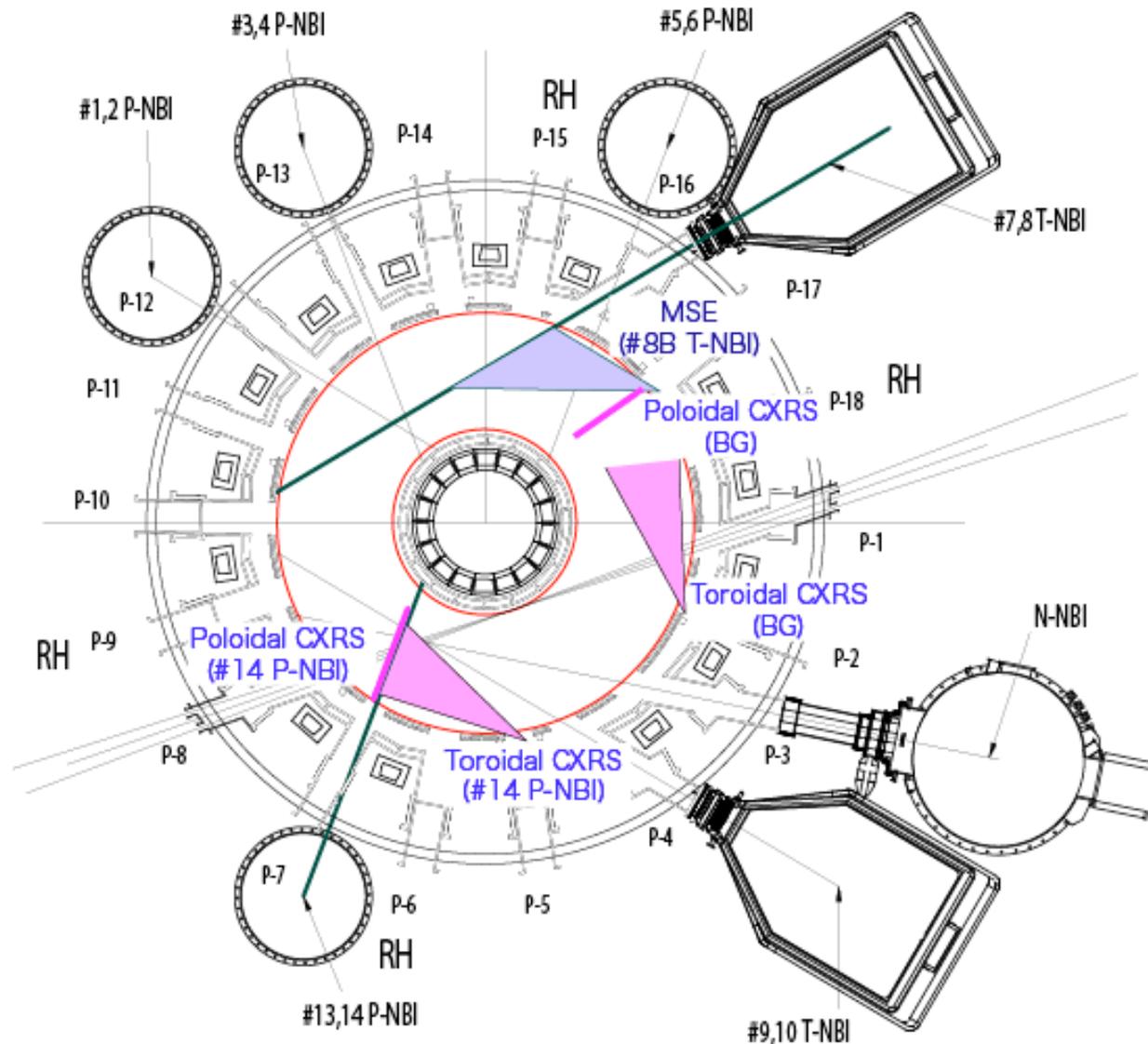
- A large enhancement of the spatial resolution of YAG laser Thomson scattering from JT-60U is required for  $n_e$  and  $T_e$  profile measurement.
- The enhanced electron enhanced heating capabilities ( $P_{NNB} = 10$  MW,  $P_{ECH} = 7$  MW), requires the YAG Thomson scattering system capabilities to measure  $T_e = 30$  keV.

P-5&P-2 Toroidal field of view, P-8&P-17 Poloidal field of view  
Toroidal & Poloidal CXRS (charge exchange recombination spectroscopy)

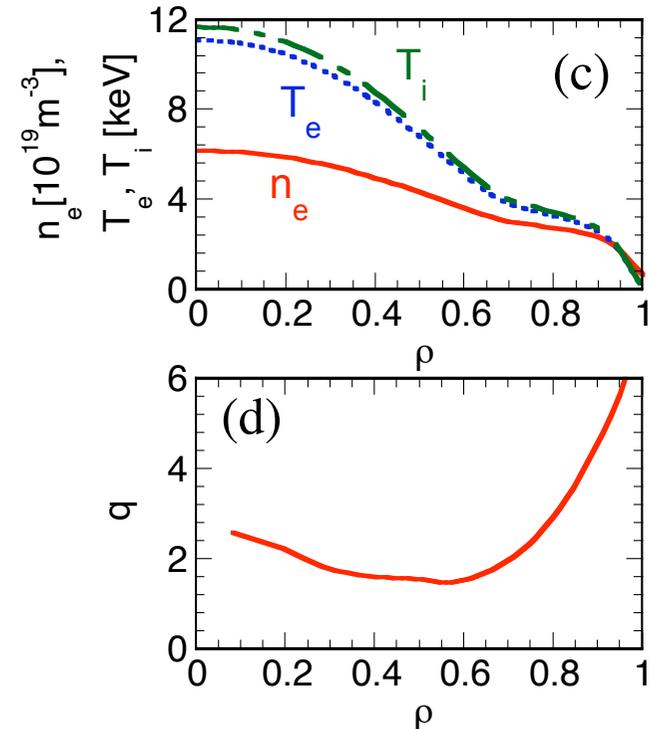
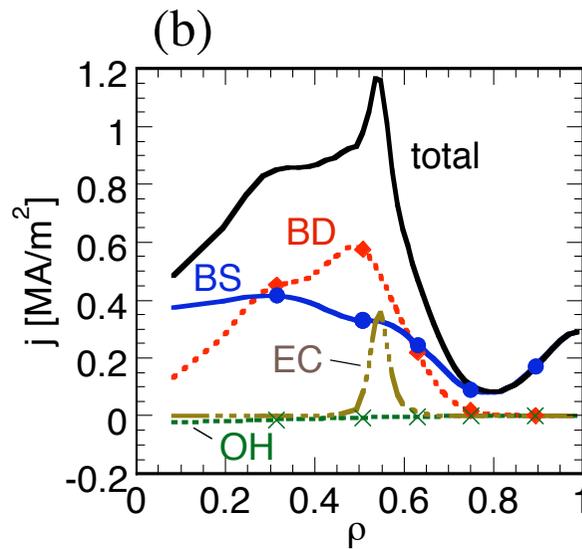
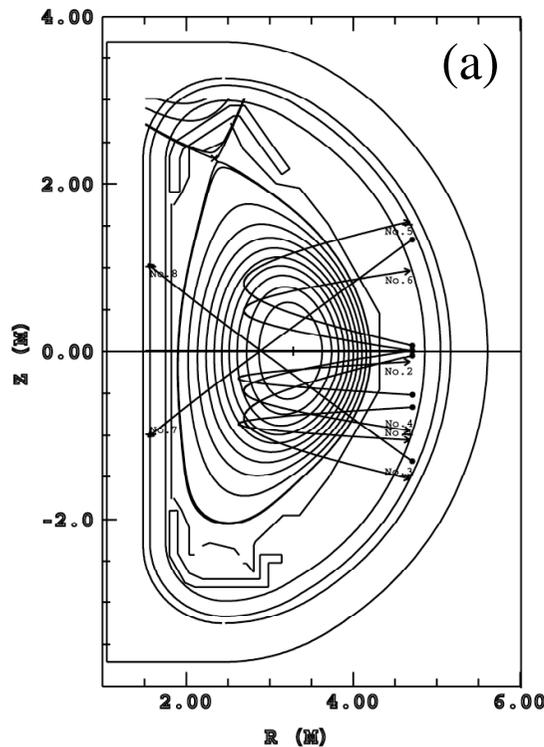


P-17 Tangential field of view  
MSE (MSEortional Stark effect polarimeter)

JT-60SA



- ECH has very localized absorption and current drive profile in the middle of the plasma.
- A spatial resolution of  $d/a \sim 0.03$  is required in MSE measurement. (Improvement in spatial resolution by a factor of 1.5 from JT-60U )

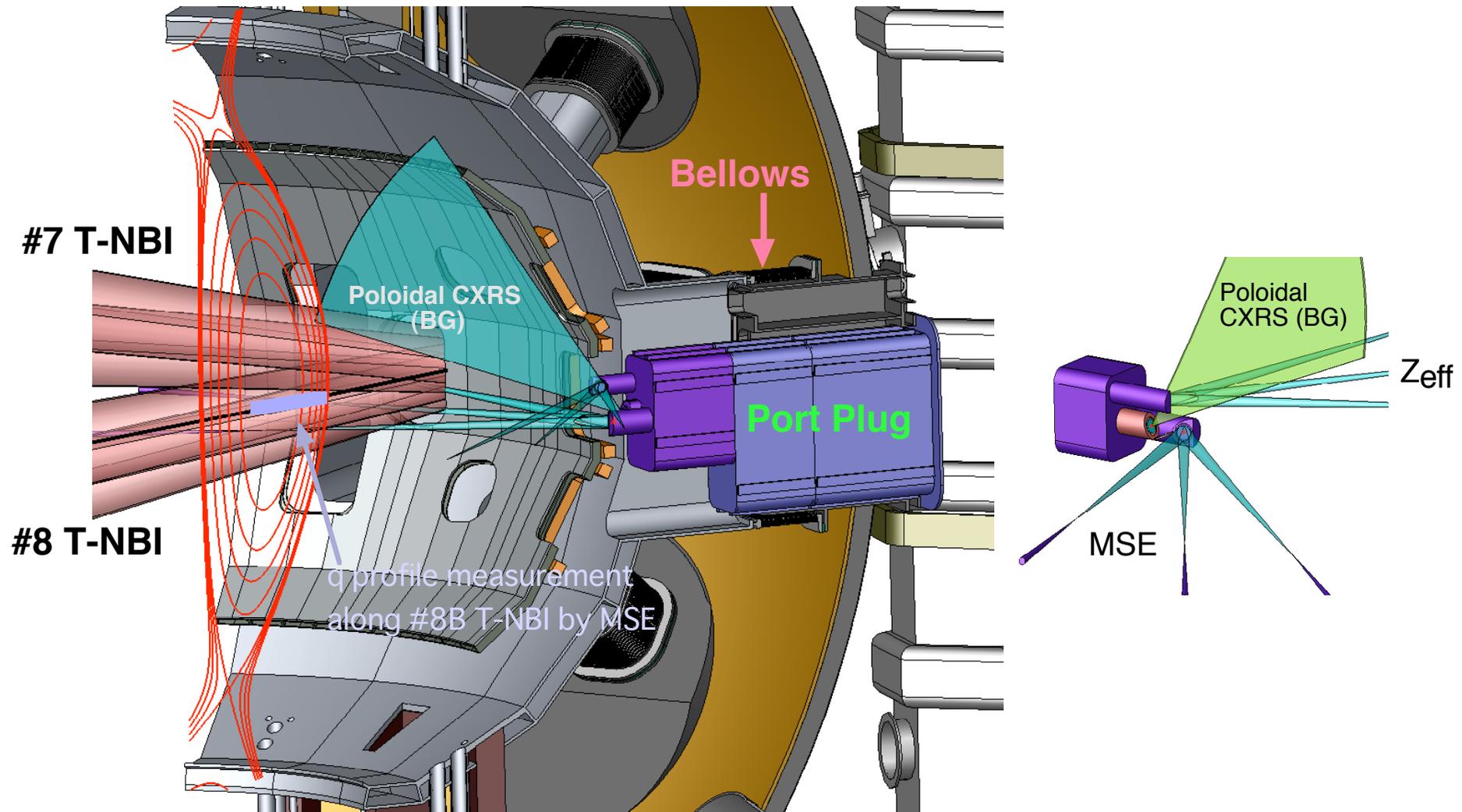


*A full CD plasma scenario, produced by ACCOME code in the conditions with  $I_p = 3$  MA ( $B_t = 2.27$  T,  $q_{95} = 5.6$ ,  $b_N = 3.6$ ,  $b = 4.2\%$ ,  $f_{BS} = 0.55$ ) for  $H_{H98y2} = 1.33$  with total heating power of 41 MW.*

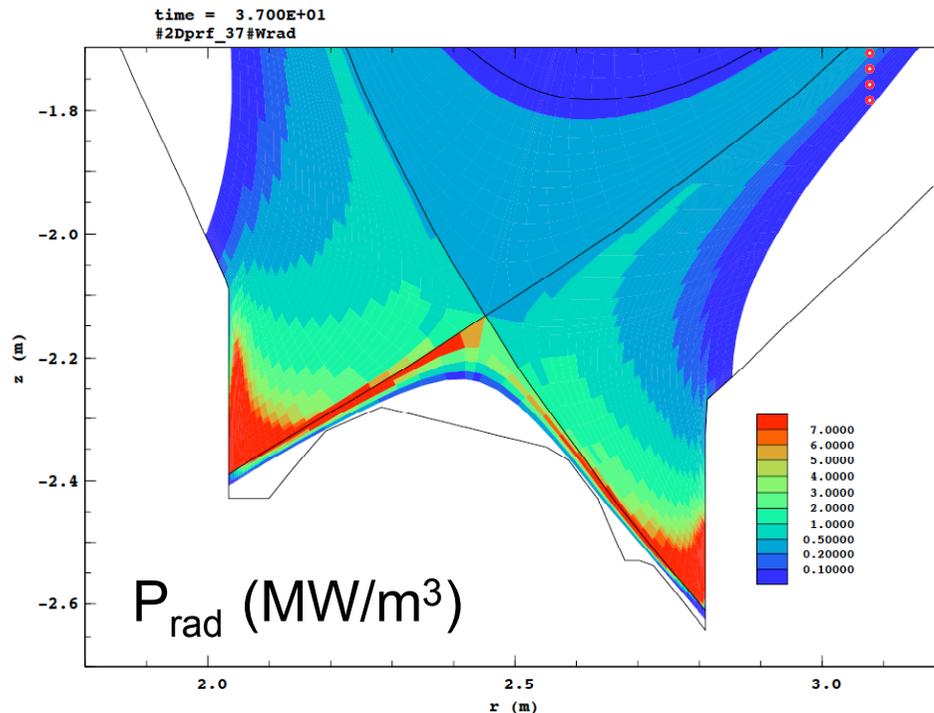


The objective optics of MSE,  $Z_{\text{eff}}$  and CXRS are enclosed in a port plug and inserted in P-17 horizontal port section

JT-60SA



- Detached divertor associated with strong flow in SOL is the key physics issue for the heat and particle control for steady state operation.
- The peak heat flux density must be suppressed below 15 MW/m<sup>2</sup> by the enhanced radiation power near the strike point of separatrix under the full heating power of 41 MW in JT-60SA.



- $D\alpha$ ,
- Divertor visible
- Divertor VUV
- Bolometer
- Divertor IR camera
- Langmuir probes
- YAG laser Thomson scattering

*Simulation with the plasma fluid code (SOLDOR) and the neutral Monte-Carlo code.*

# Divertor optical diagnostics

Visible & VUV spectrometers for divertor,  $Z_{\text{eff}}$  monitor

- Upper divertor viewed from the lower port: Lower divertor viewed from the upper port:

