

High Beta Plasma Research

Presented by T. Ozeki

DOE-JAERI Technical Planning of Tokamak Experiment
and IEA Large Tokamak Workshop on Experimental Planning
JAERI, Naka
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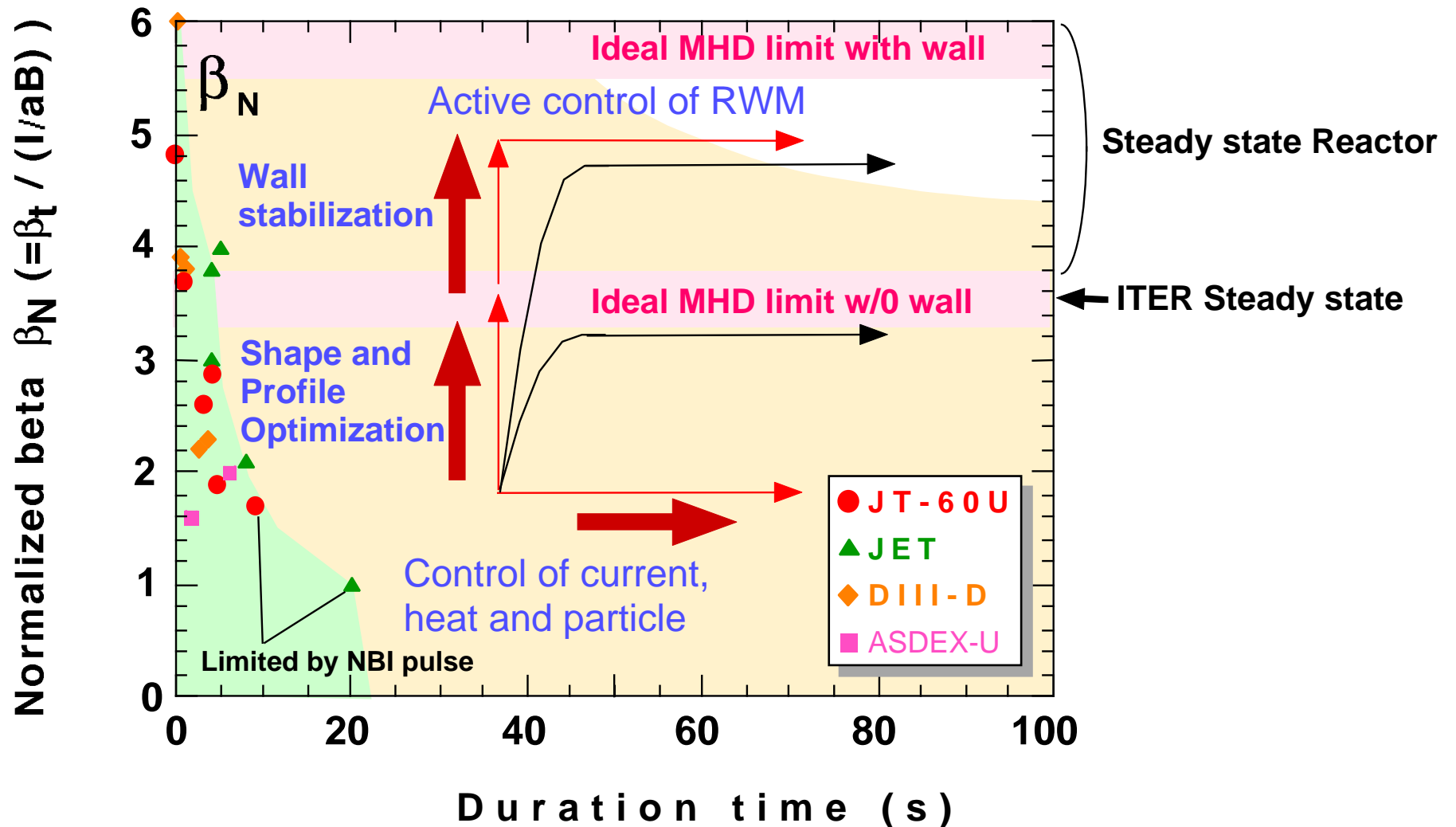
Objectives

Establishment of high beta plasmas in steady-state by

- optimization of profiles of pressure, current, rotations, etc.,**
- optimization of plasma shaping,**
- establishment of active plasma control for a compact (or effective) reactor.**

Extend to Steady-State, High Beta Regime

JT-60SC

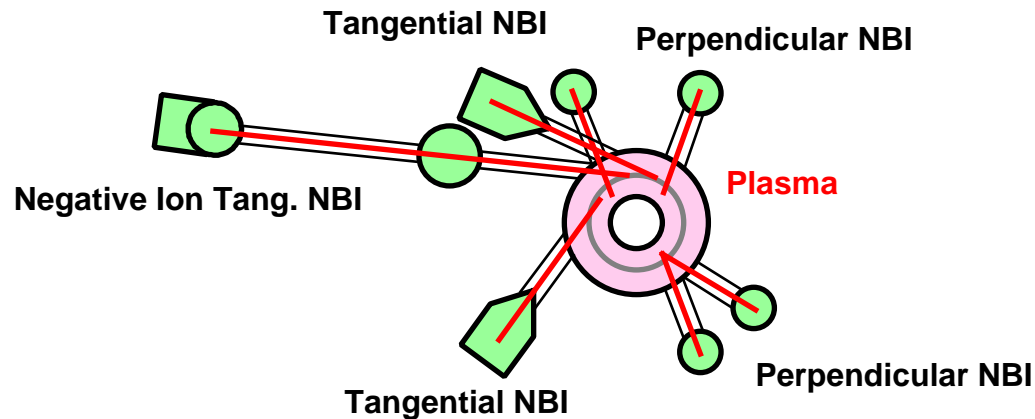


Production and Sustainment of High beta plasma

JT-60SC

To obtain and sustain high beta plasma, high power heating and current drive system of JT-60 is utilized.

	10 sec	20 sec	30 sec	50 sec	100sec
Perpendicular NBI	20MW	20MW	13.3MW	13.3MW	6.7MW
Tangential NBI	10MW	10MW	6.7MW	6.7MW	3.3MW
Negative Ion Tang. NBI	10MW	7MW	7MW	3MW	3MW
ECH	4MW	3.75MW	3.1MW	2.4MW	1.7MW
Total	44MW	40.75MW	30MW	25.4MW	14.7MW



Top view of JT-60SC

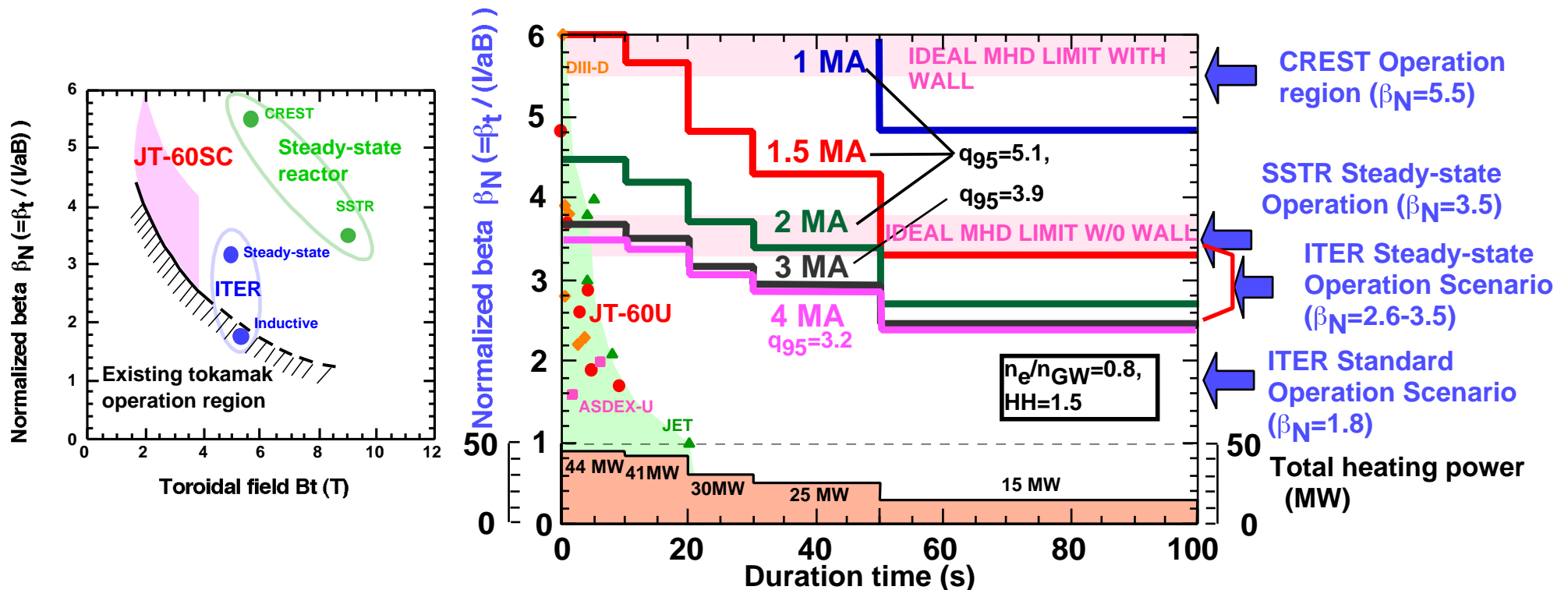
Investigate New High Beta Regimes

JT-60SC

Very High β : $\beta_N \sim 6$, 10sec, 1.5MA, $\beta_t = 2.1T$, $\tau_{li}/R \sim 15$ sec

High β in SS : $\beta_N \sim 3$, 100sec, 2MA, $\beta_t = 2.8T$, $\tau_{li}/R \sim 20$ sec

High β in High B_t : $\beta_N \sim 3.0$, ~ 30 sec, 4MA, $B_t = 3.8T$, $q_{95} = 3.2$, $\tau_{li}/R \sim 40$ sec



Goal: Establish high beta-N plasma

- during non-inductive full CD with large bootstrap current fraction,
- in a steady-state where the duration time exceeds a current diffusion time.

Approach to high beta regime in steady-state and high Bt

- (1) Profile control
 - Current, Pressure (temperature, density), Rotation
- (2) Shape control
 - Elongation, Triangularity
- (3) Active feedback control
 - Neoclassical tearing mode, Resistive wall Mode

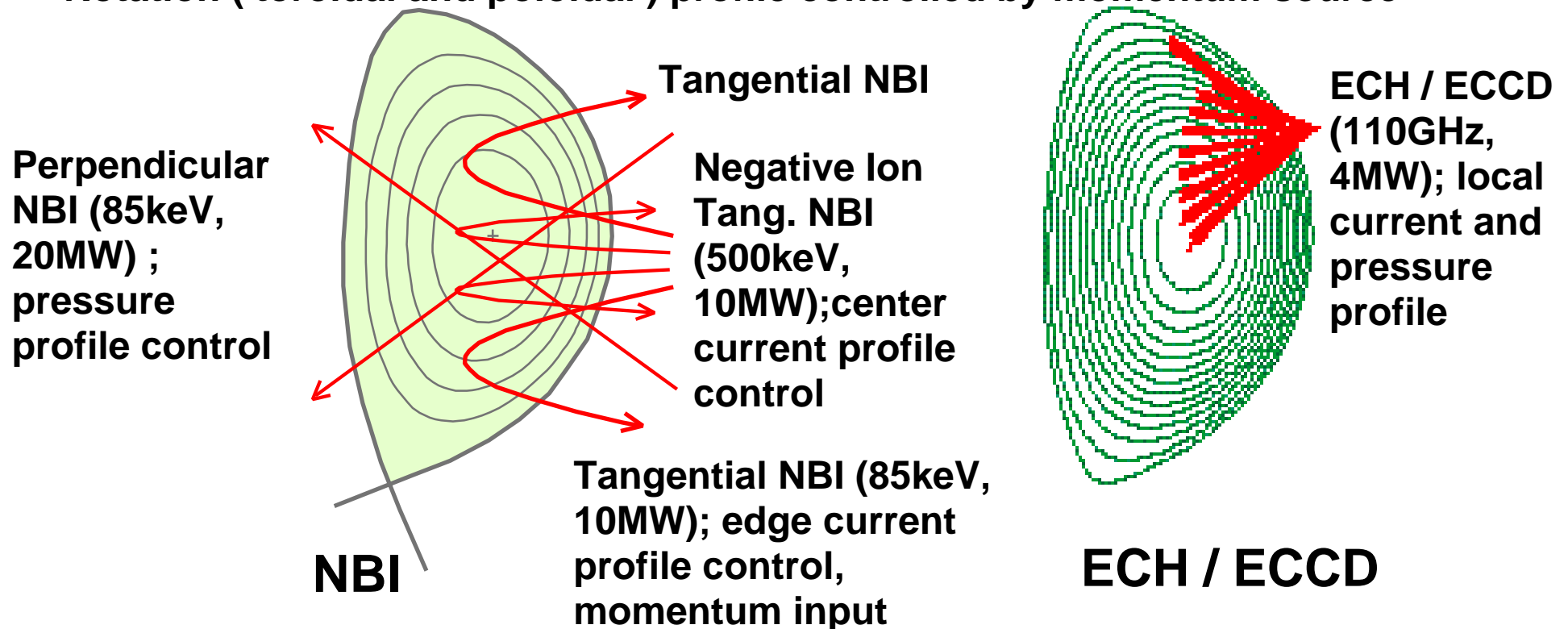
(1) Profile Control

JT-60SC

ISSUES: In plasmas with large bootstrap fraction and high confinement such as ITB, pressure and current profiles are strongly coupled each other .
Seek the best combination of pressure and current profiles.

JT-60SC has various facilities for profile control;

- Pressure (temp. and density) profile controlled by heating and particle source
- Current density profile controlled by driven current and bootstrap throu. pressure
- Rotation (toroidal and poloidal) profile controlled by momentum source



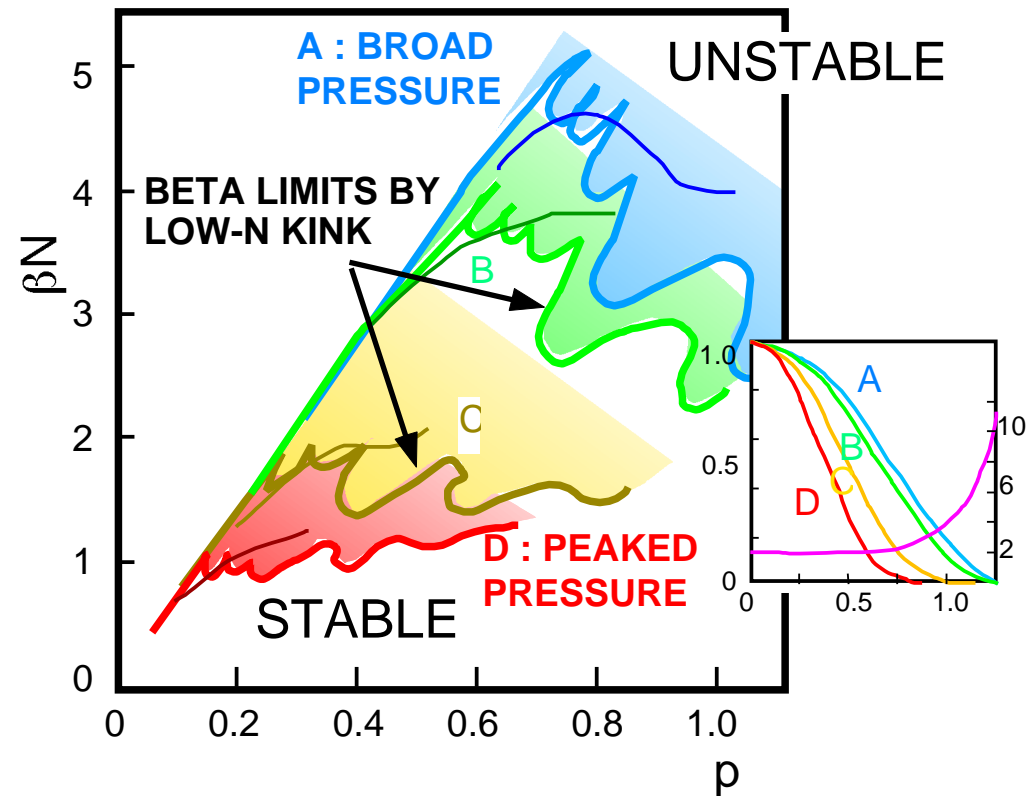
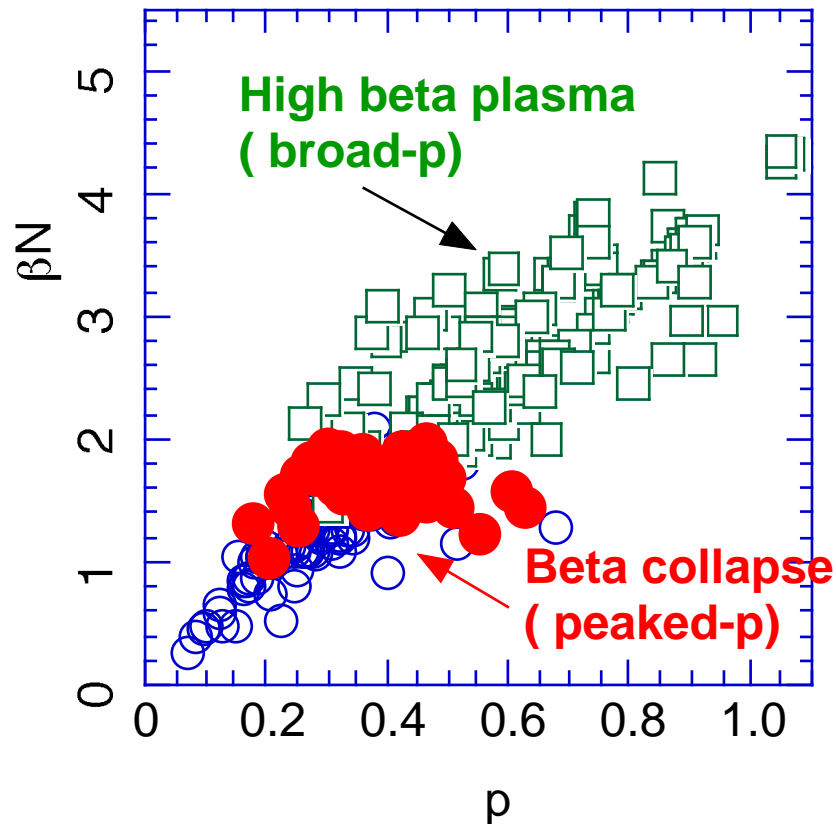
High beta-p operation

JT-60SC

High beta-p plasmas (positive shear) in JT-60U;

Direction to good stability:

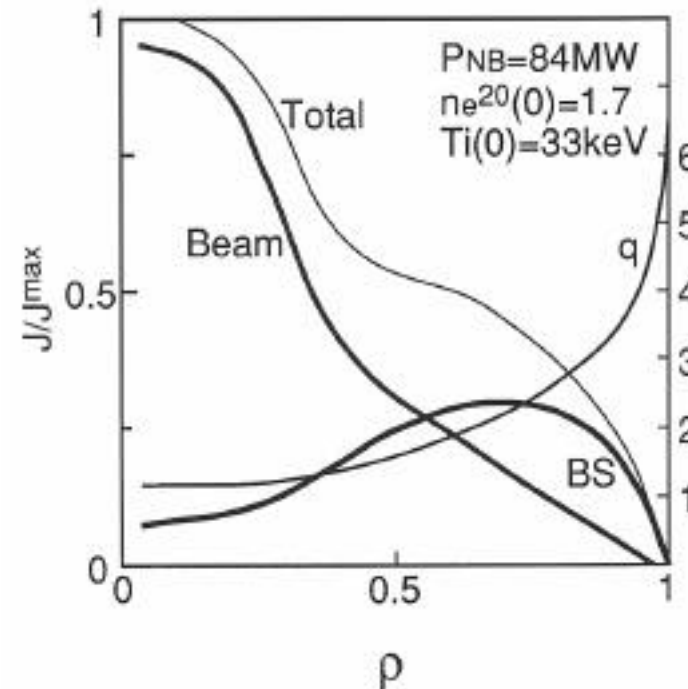
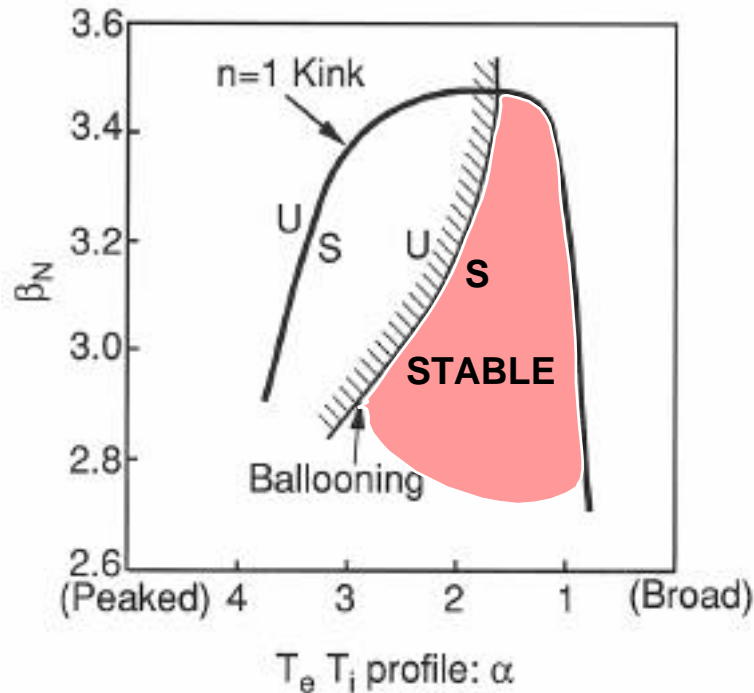
- Broad Pressure profile are good for stability.
- Current density with high shear is good for low-n stability, and low and negative shear is good for high-n ballooning stability.



Self consistent analysis: High beta plasmas ($\beta_N < 3.5$) without wall obtained by profile optimization

JT-60SC

Self consistent equilibrium of steady-state, high beta plasma in SSTR configuration



$n_e, n_i \propto (1-\rho^2)^{0.5}$, $T_e, T_i \propto (1-\rho^3)^\alpha$, SSTR Scenario

Monotonic q-profile, low-n kink stable, 1st stable to ballooning mode for $r_{wall} / a = \infty$. fbs=50%

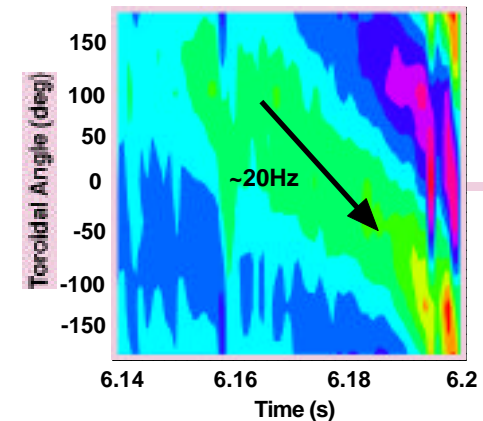
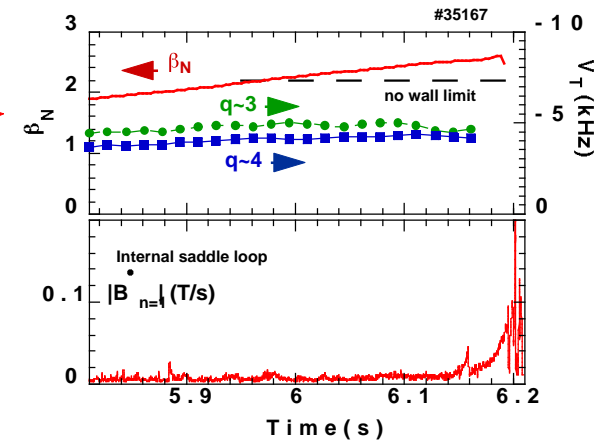
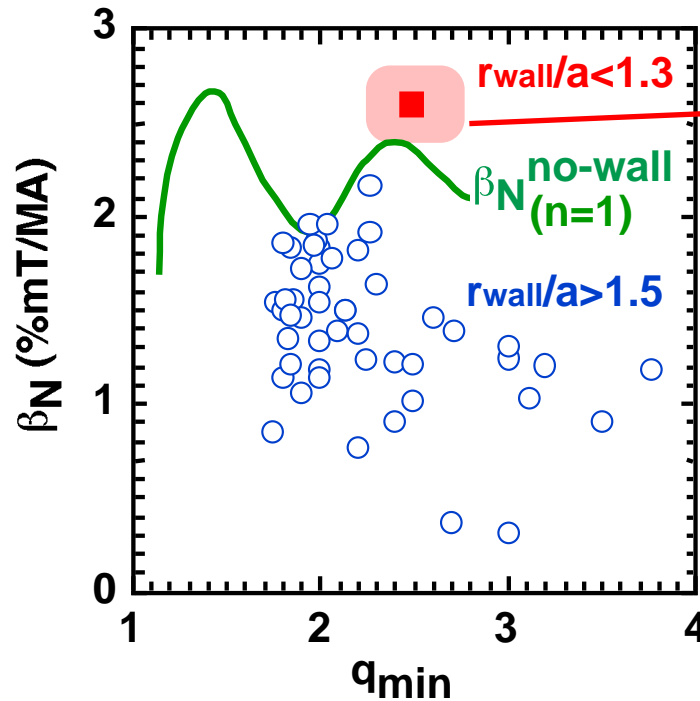
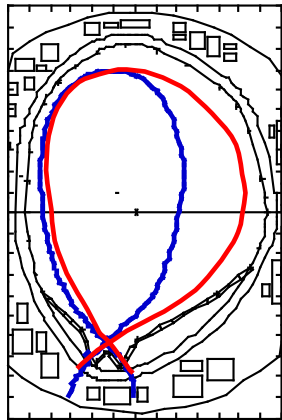
Key issue: Self-consistent profile optimization
Stabilization of neoclassical tearing mode

Reversed shear operation

JT-60SC

Reversed shear plasmas is promising for steady state with large bootstrap current. In JT-60U, High QDT(=1.25) was obtained but terminated by the beta collapse.

- $r_{wall}/a > 1.5$:
 - Achievable β_N is limited at the **no-wall** ideal stability boundary
 - H-mode edge plasma is better stability than L-mode edge.
- $r_{wall}/a < 1.3$:
 - Achievable β_N is improved to be **above the no-wall limit**.
 - Improved β_N discharges are terminated with appearance of RWM.

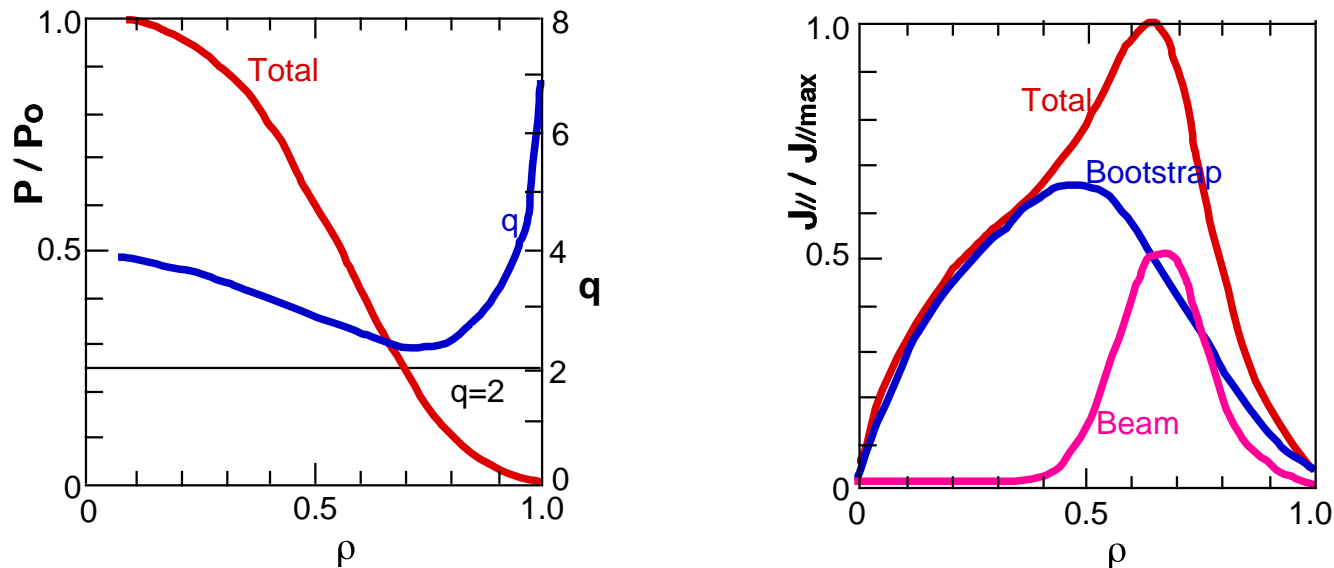


- $n=1$ with growth time $\gamma^{-1} \sim 10^{-2} \text{ s} \sim \tau_{wall}$ ($\tau_{wall} \sim 10 \text{ ms}$)
- Rotating in the toroidal direction with $f \sim 20 \text{ Hz} \sim 1/(2\pi\tau_{wall})$
- No clear reduction was observed in the toroidal rotation frequency

Reversed Shear, High beta plasmas obtained by the profile optimization and wall stabilization

JT-60SC

Self consistent reversed shear equilibrium of steady-state, high beta plasma with large bootstrap current in SSTR configuration



$\beta_N=3$, $\beta_p=1.6$, $f_{bs}=65\%$, SSTR scenario

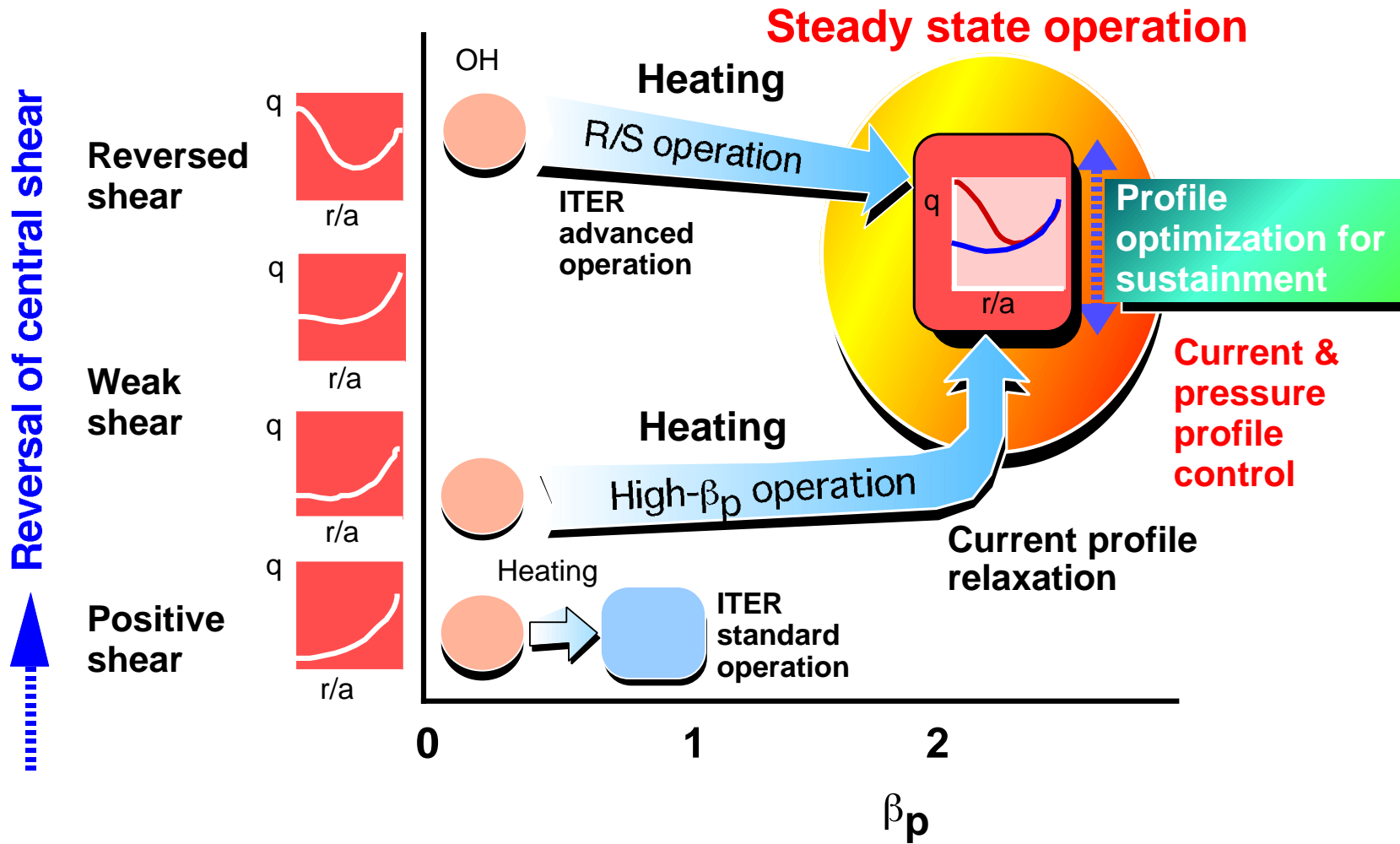
Low-n kink stable, 1st stable to ballooning mode for $r_{wall} / a = \infty$

β_N increases up to >4 and f_{bs} up to $>80\%$ by the wall stabilization: $r_{wall}/a=1.2$.

Key issue: Sustainment profile optimization
Stabilization of resistive wall mode

Profile Optimization toward Steady State Operation

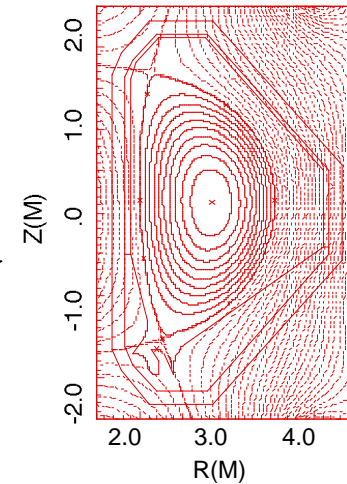
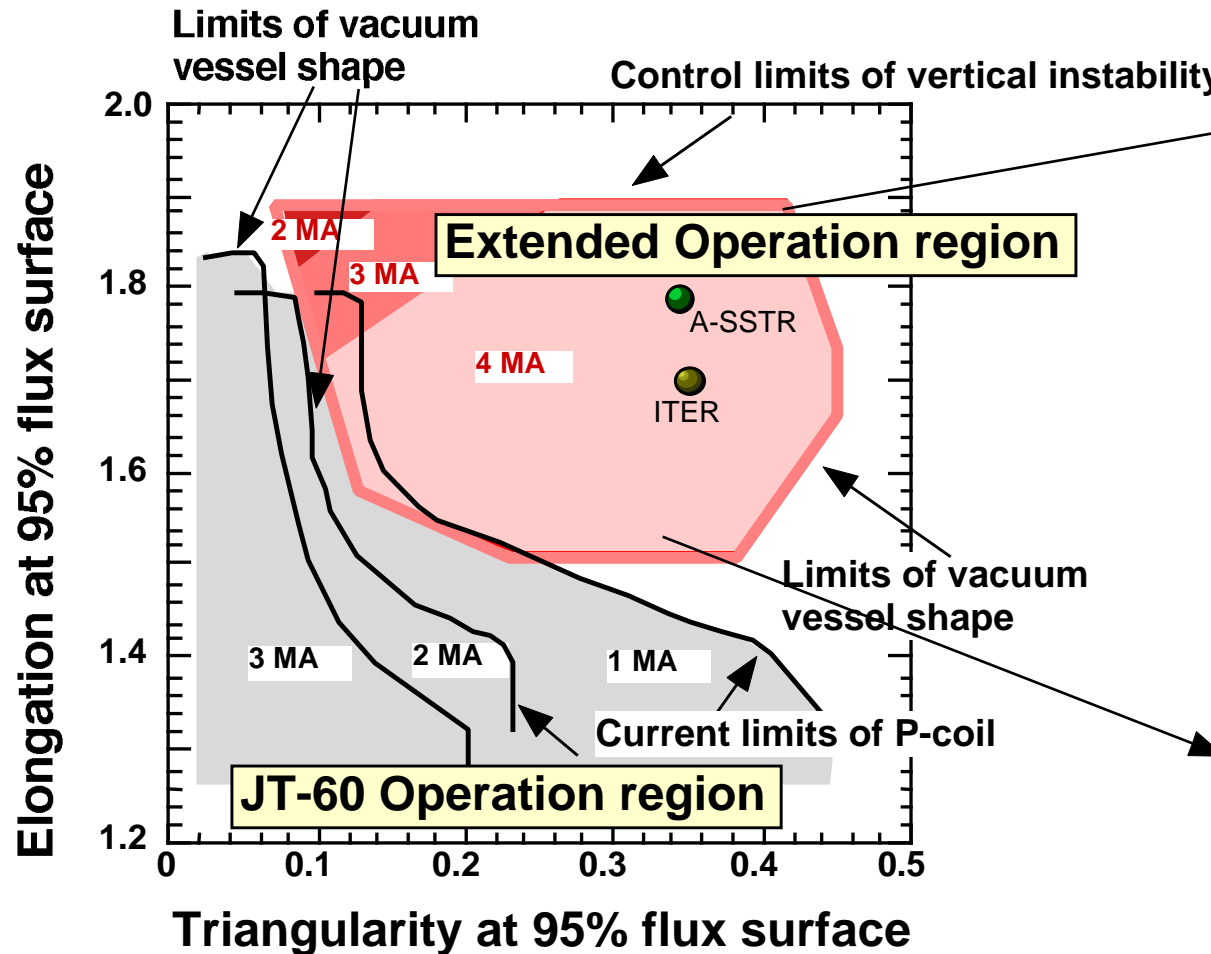
JT-60SC



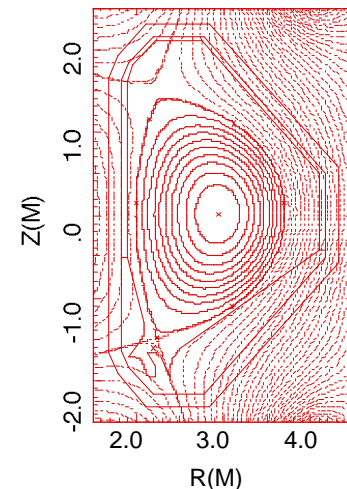
(2) Shape Control

JT-60SC

ISSUES: Investigate effects of elongation and triangularity on the edge stability and the confinement. Optimization of both shapes and profiles.



Limited by vertical instability.



Limited by vacuum vessel.

Extend to high elongation and high triangularity region
Consistency to the divertor is under consideration.

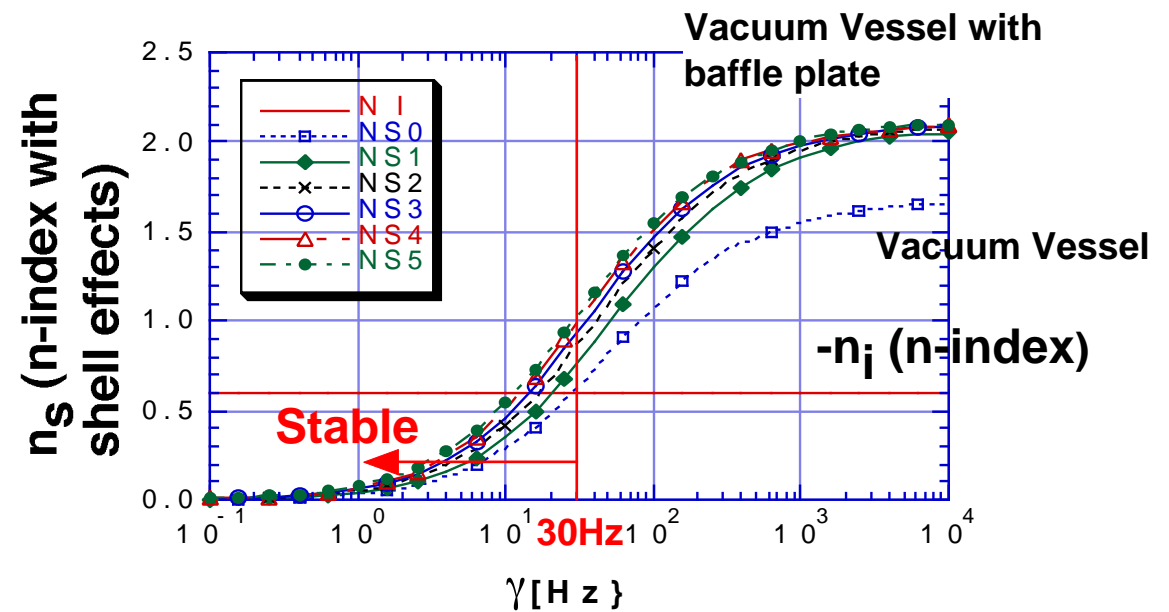
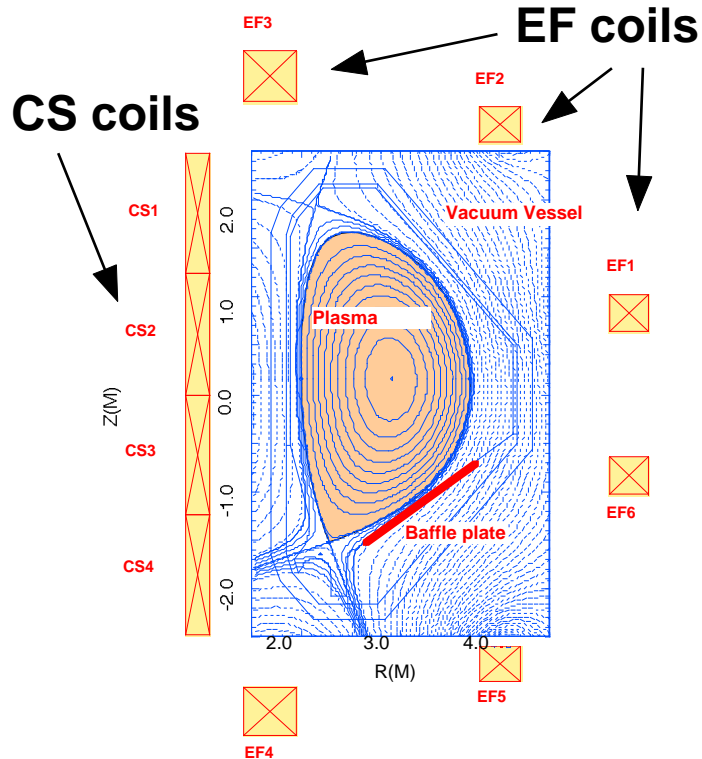
$l_i=0.7\sim 0.85$, $p(0)/\langle p(r) \rangle \sim 2.5$,
 $\beta_p=0.6\sim 1.2$, Separatrix is not on the dome.

Vertical instability

JT-60SC

$l_i=0.5, \beta_p=1.2, k_{95}=1.8, \delta_{95}=0.35$

Growth rate is reduced by baffle plate.



Growth rate less than 30Hz is stabilized by poloidal coils (CS coils and EF coils).

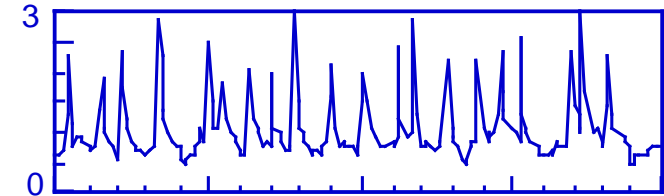
Vertical instability is stabilized under $k < 1.9$

Development of suitable ELM by plasma shaping

JT-60SC

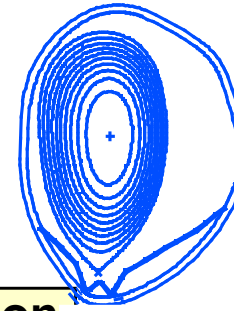
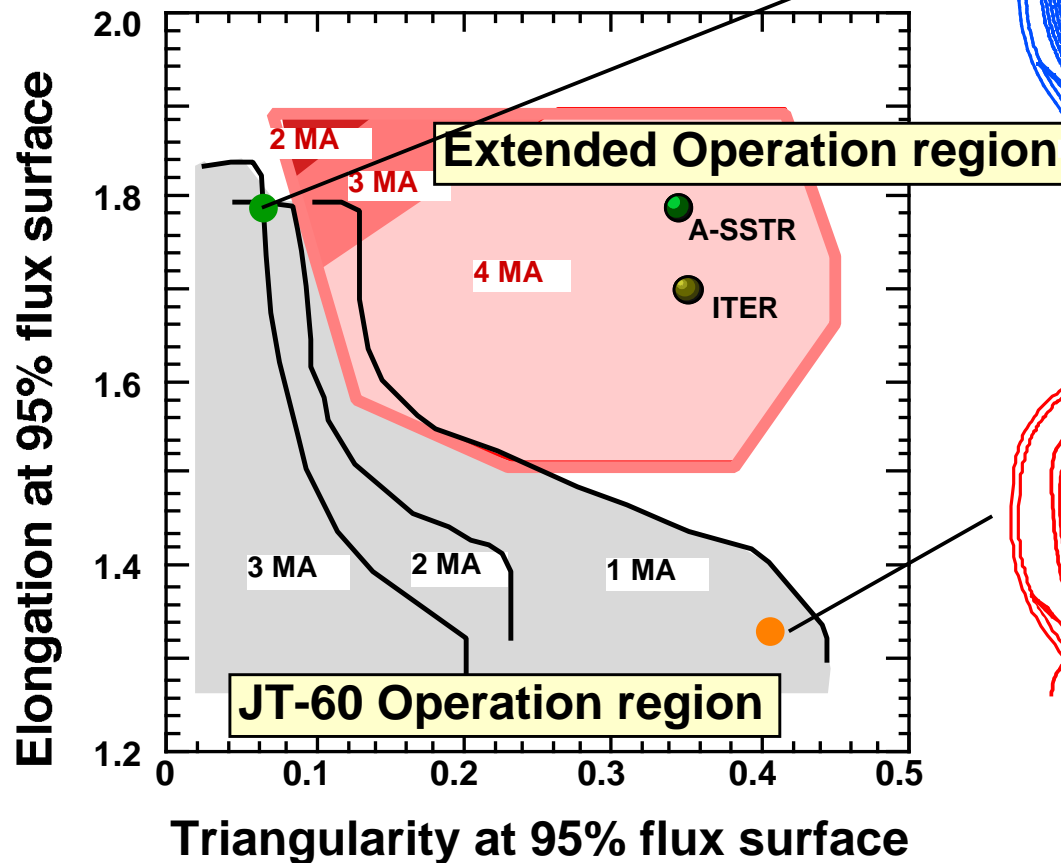
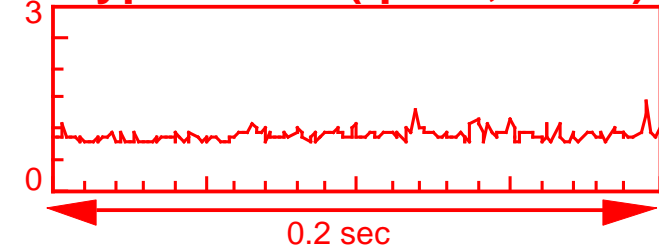
Large heat flux to Divertor

Type I ELM ($q=3.9, \delta=0.2$)



Moderate heat flux

Type II ELM ($q=6.1, \delta=0.5$)

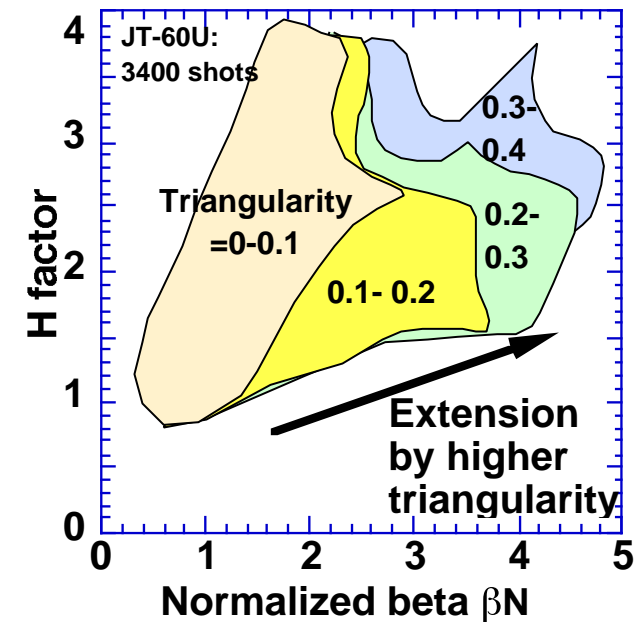
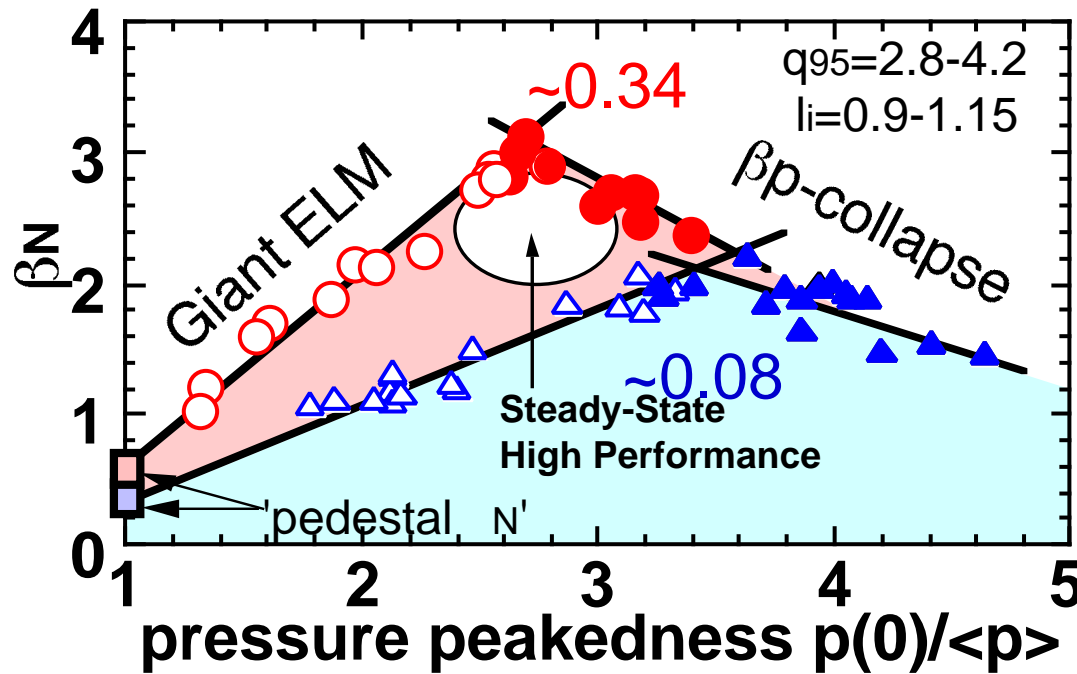


In JT-60U, the high triangularity demonstrated the moderate ELM.
In the plasma edge, plasma may exist in the 2'nd stability of ballooning mode.

Synergistic effects of shape and profile optimization

JT-60SC

The highest β_N is obtained at a medium peakedness of $p(r)$:
 : ELM & β_p -collapse (low-n ideal kink-ballooning)



At high- δ : β_N increases by $\sim 50\%$.

The optimum $p(0)/\langle p \rangle$ becomes smaller.

(3) Active feedback Control

JT-60SC

- **NTM stabilization is key issues for positive shear plasma:
Identify Islands by ECE, magnetic probe, etc
Stabilize by ECCD/ECH**
- **RWM stabilization is key issues for high beta plasma
above the ideal beta limit without wall:
Identified by sensor coils inside Vacuum Vessel
Stabilize by feedback coils inside Vacuum Vessel**
- **Real time feedback control of NTM and RWM is key issue
to find a solution of Steady State and High Beta plasma.**

Regime of NTM unstable in JT-60SC

JT-60SC

Objectives: Establish NTM control technology for reactor relevant conditions at high beta-N during non-inductive full CD with a large bootstrap fraction.

ITER-FDR Physics R&D [Sauter]

$$N \approx 5.2 e^{*0.3}$$

JET [IAEA, Yokohama 1998]

$$N \approx e^{*-0.2 * 0.6}$$

DIII-D [Lahaye 2000]

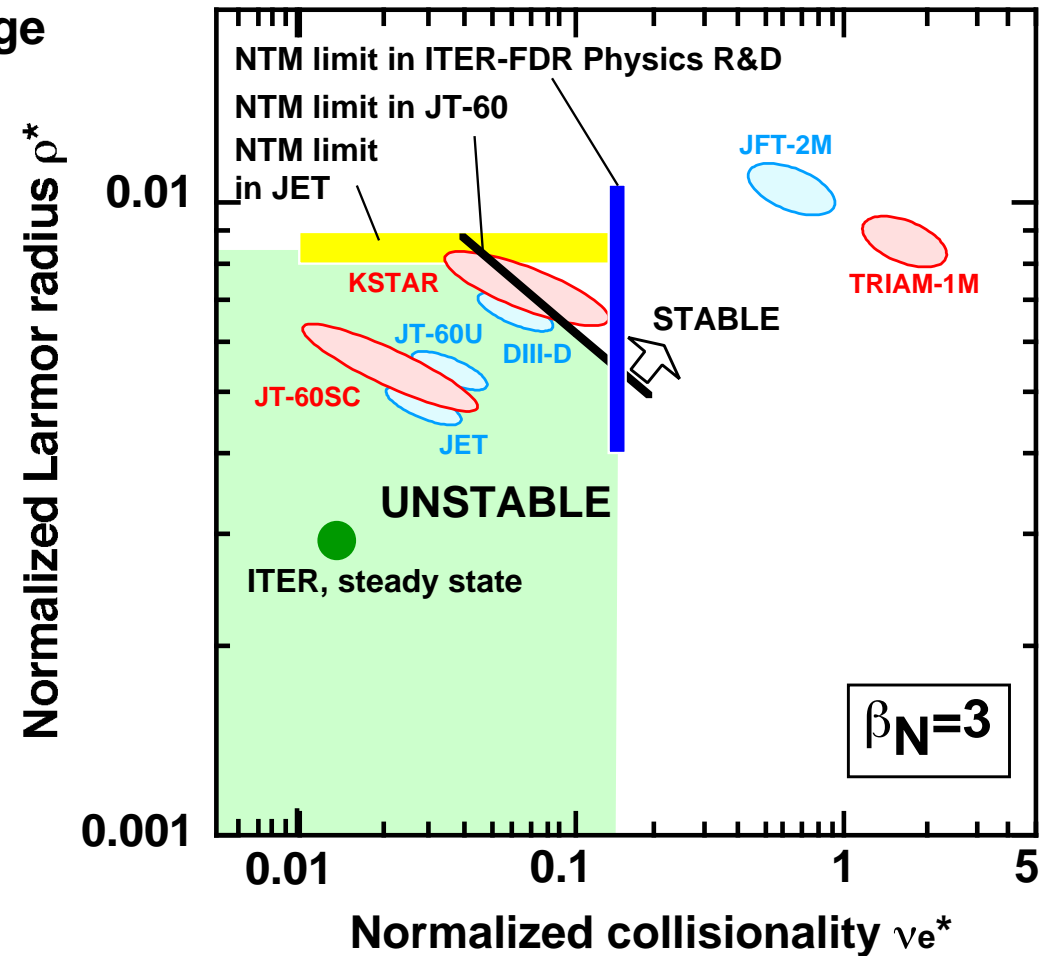
$$N \approx (i^*/e^*)^{0.43 * 1.3}$$

ASDEX [Lahaye 2000]

$$N \approx (i^*/e^*)^{0.25 * 1.1}$$

JT-60 [IAEA, Sorrento 2000]

$$N \approx e^{*0.4 *}$$

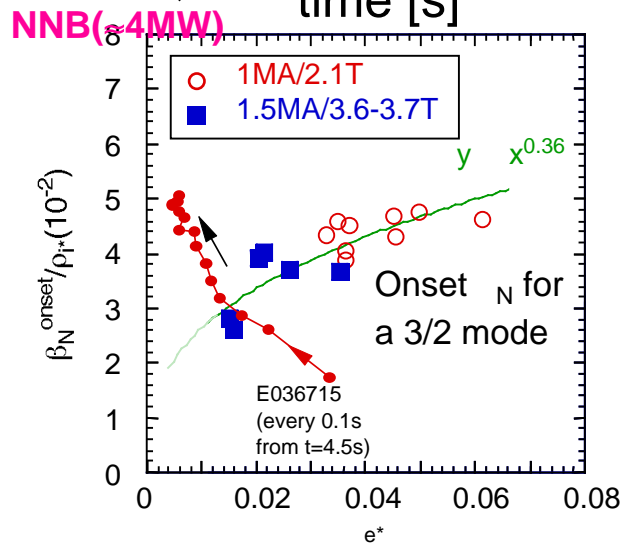
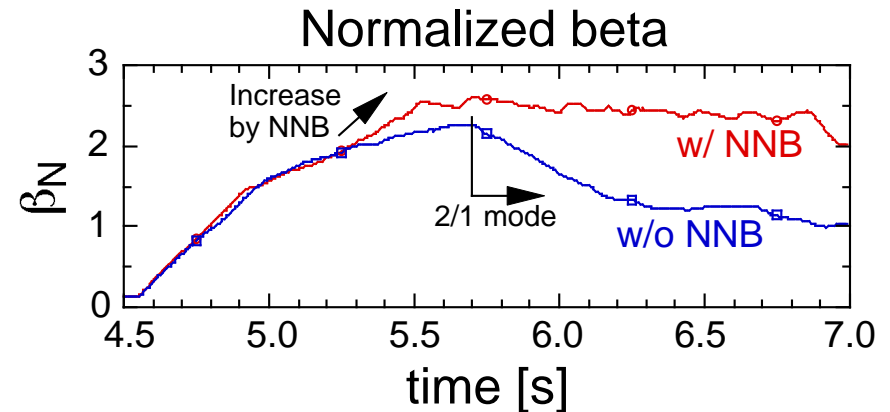
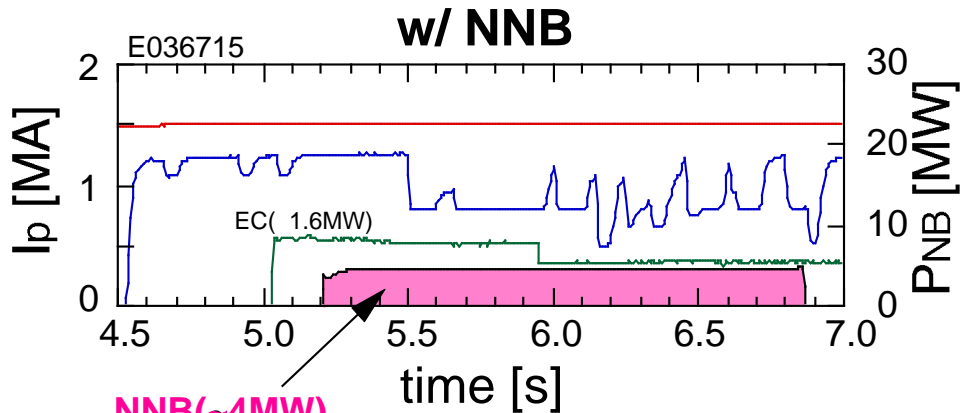


Onset βN of JT-60SC is $\sim 1-2.2$
($\nu_e^* \sim 0.01-0.05$, $\rho^* = 0.004-0.005$)

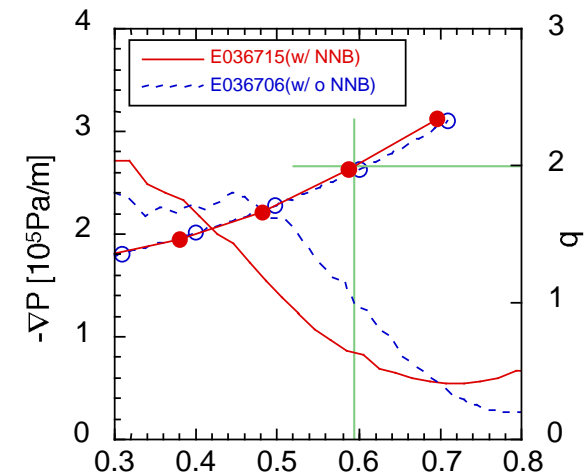
No tearing mode was observed in discharge with NNB

JT-60SC

- Beta value is exceeded the onset beta scaling with NNB.
- Without NNB, a 2/1 mode appears at t=5.7s.



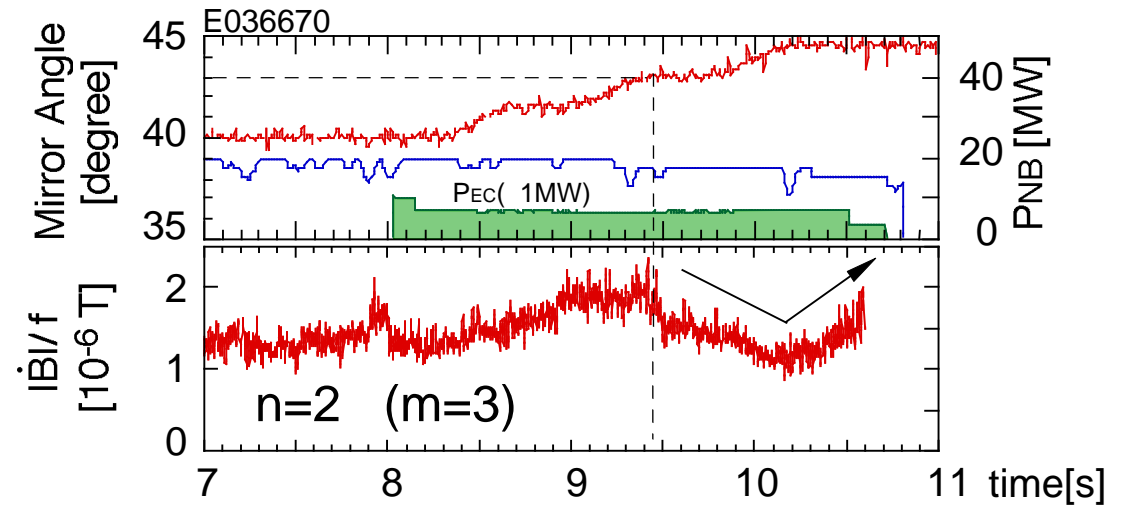
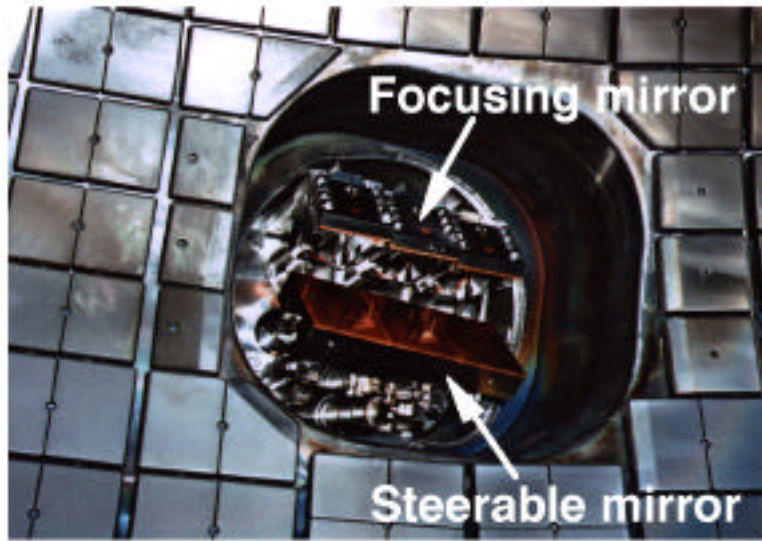
Reduction of pressure gradient at the mode rational surface may improve the stability for NTM.



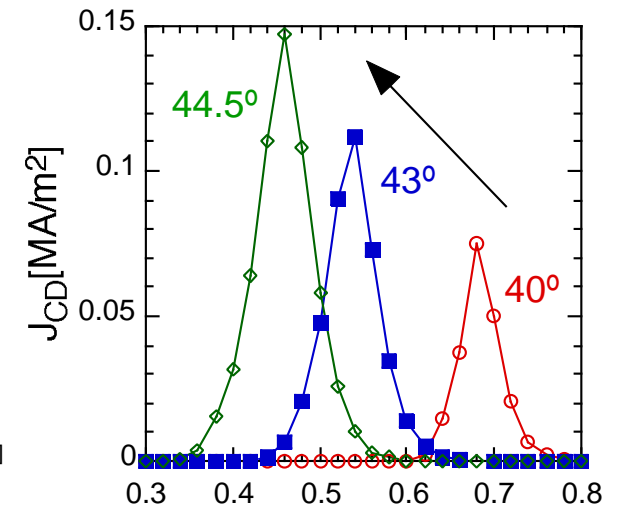
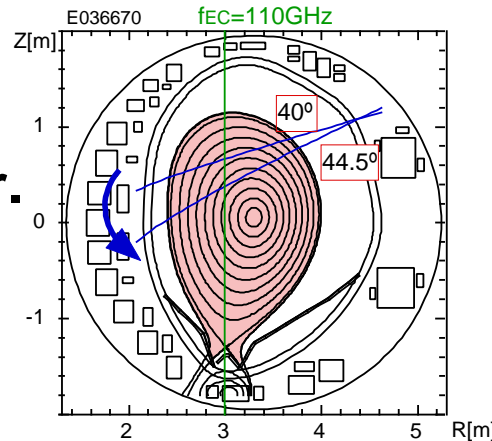
Key issue: Investigation for profile optimization for the NTM suppression.

NTM Stabilization by ECCD System

JT-60SC



- Design value: 110GHz; 1 MW at each gyrotron output (x3)
- Poloidal injection angle can be changed by the steerable mirror.

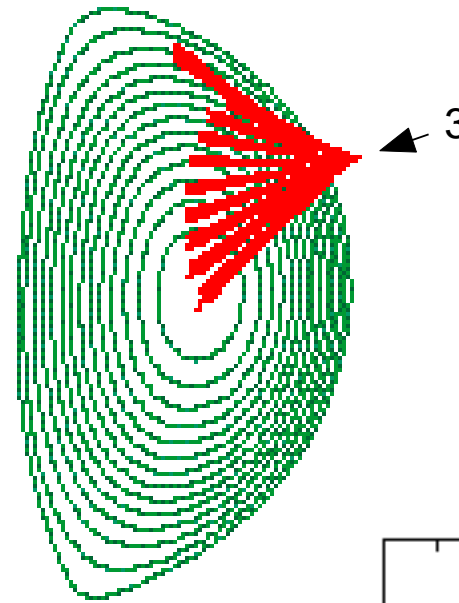
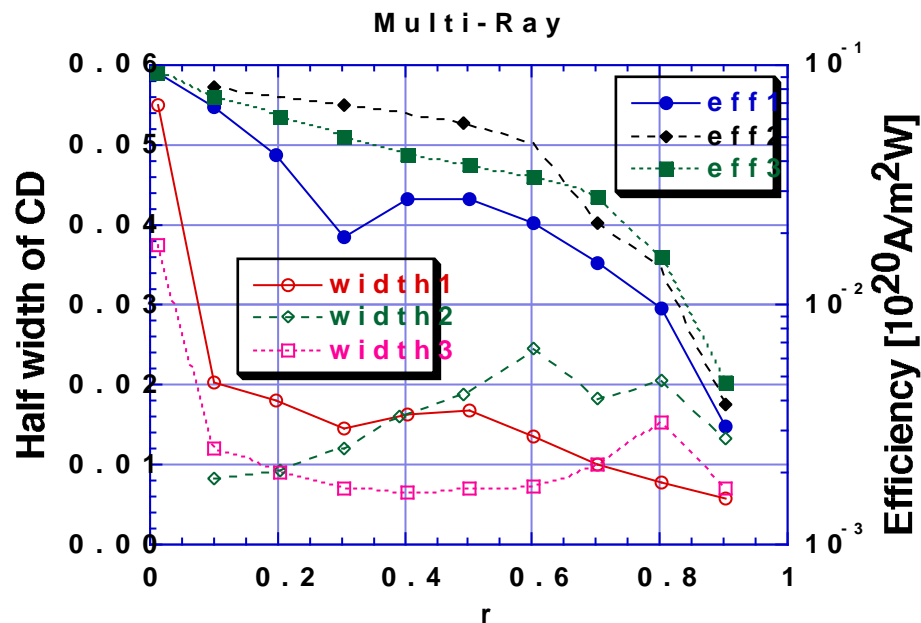
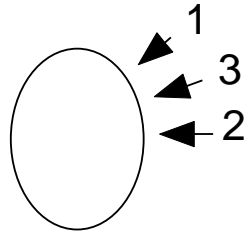


- Magnetic perturbations with $n=2$ decreases at the injection angle of 43° .
- Complete stabilization was also obtained in low beta plasma.

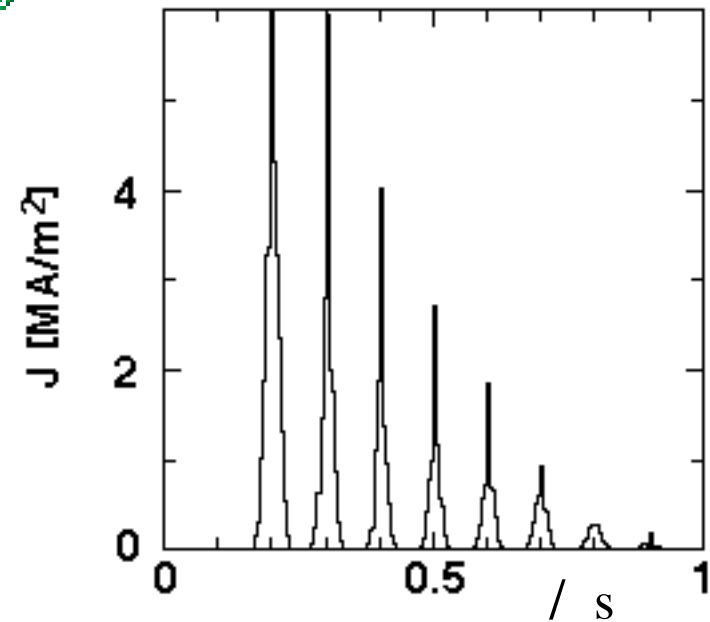
EC Current Drive Efficiency

JT-60SC

Evaluation of current drive efficiency by ray tracing analyses



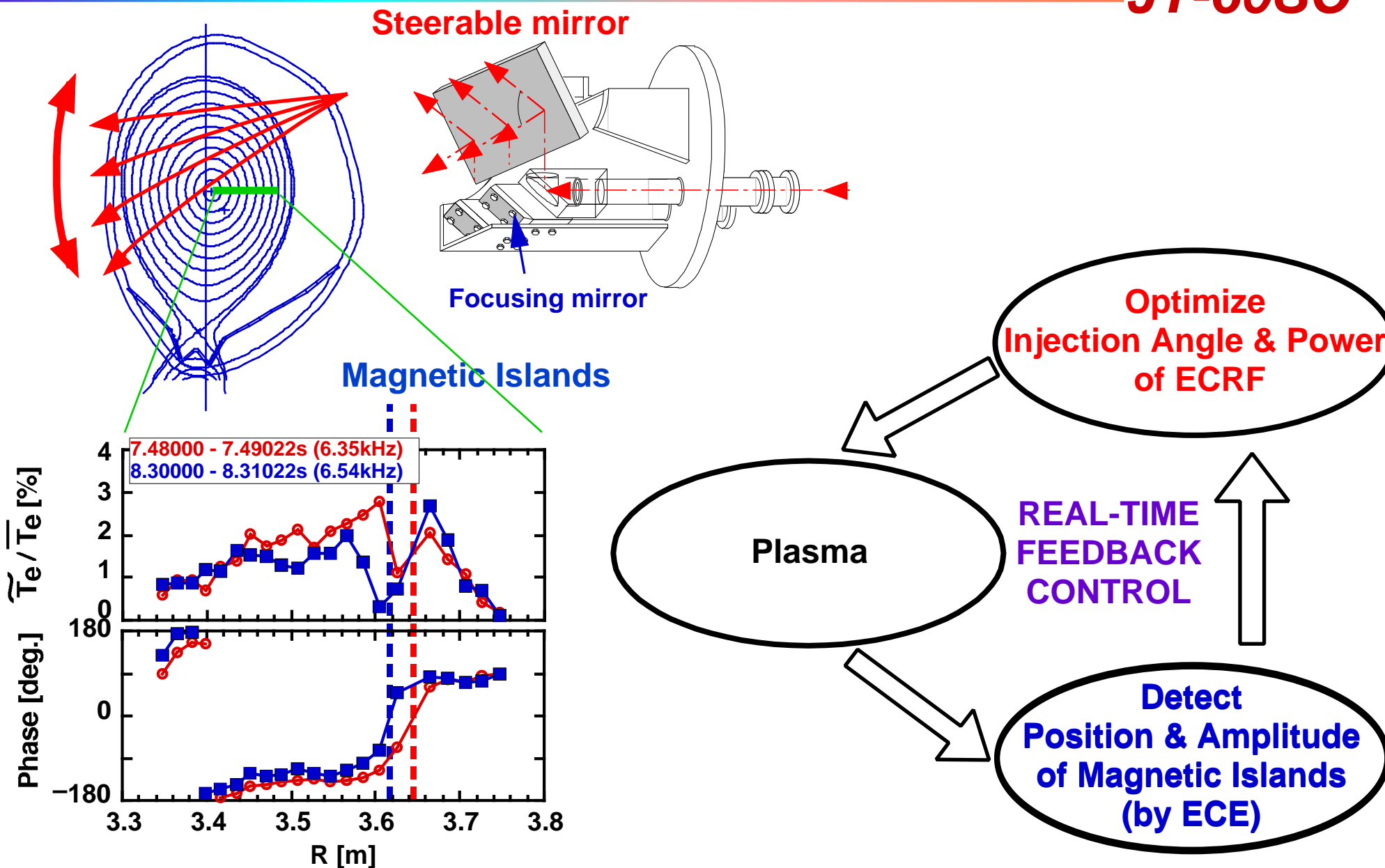
Localized driven current can be expected by the angle 3.



- Angle 3, correspond to the port, is good current drive efficiency.
- $T_{e0}=15\text{keV}$, $n_{e0}=5\times 10^{19}\text{m}^{-3}$ with parabolic profile.
- $f_{ECW}=110\text{GHz}$, fundamental resonance of O-wave.

Real-Time Feedback Control of NTMS will be Demonstrated on JT-60U to Prove Reliability of Steady State High Beta Operation on JT-60SC

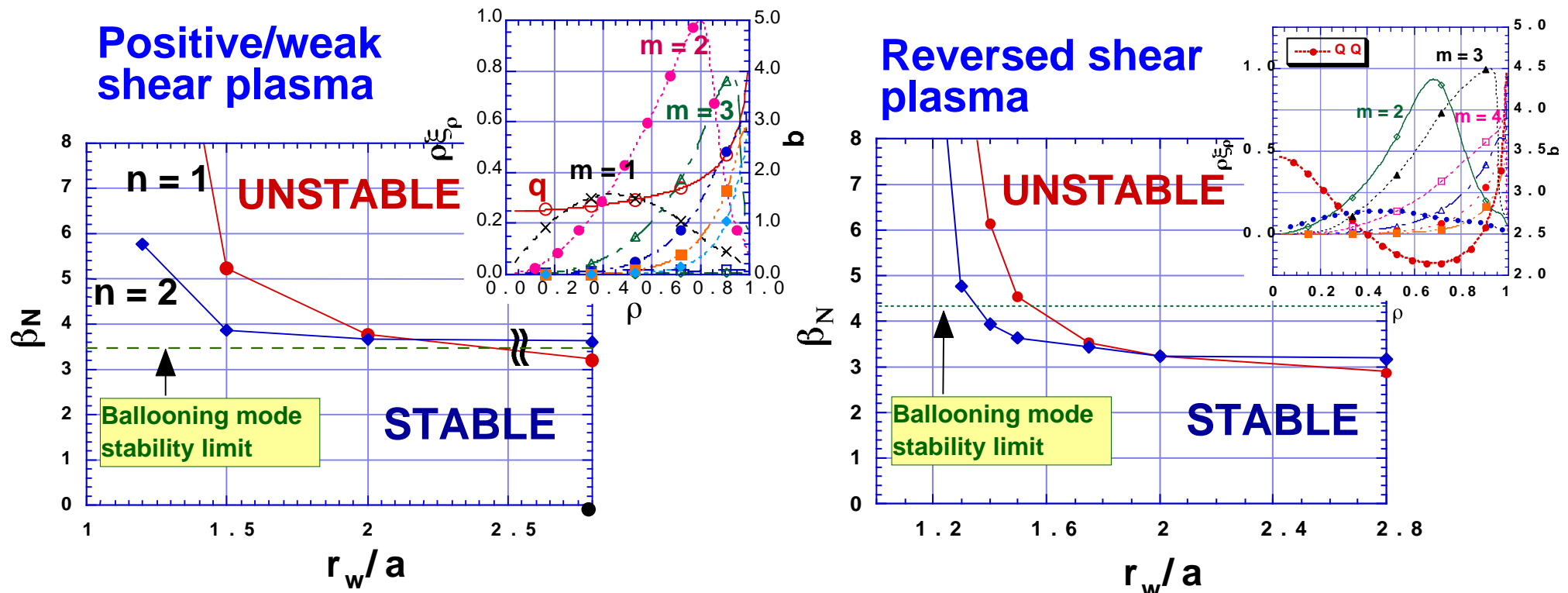
JT-60SC



Stability with the wall

JT-60SC

- **Objectives** Establish RWM control technology for reactor relevant conditions;
 - at high beta-N over ideal MHD limit w/o wall
 - during non-inductive full CD with a large bootstrap fraction



Significant Improvement of Ideal MHD Stability by the Wall

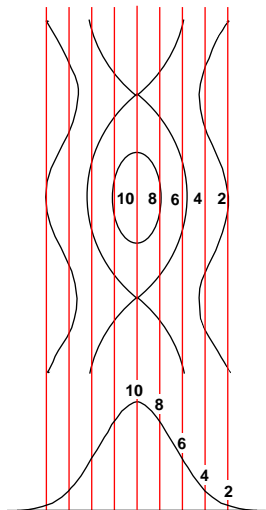
Low-n kink and high-n ballooning mode stability in JT-60SC configuration. $R_0 = 2.8 \text{ m}$, $a = 0.85 \text{ m}$, $\kappa_{95} = 1.8$, $\delta_{95} = 0.35$

Wall stabilization is pronounced in reversed shear plasmas.

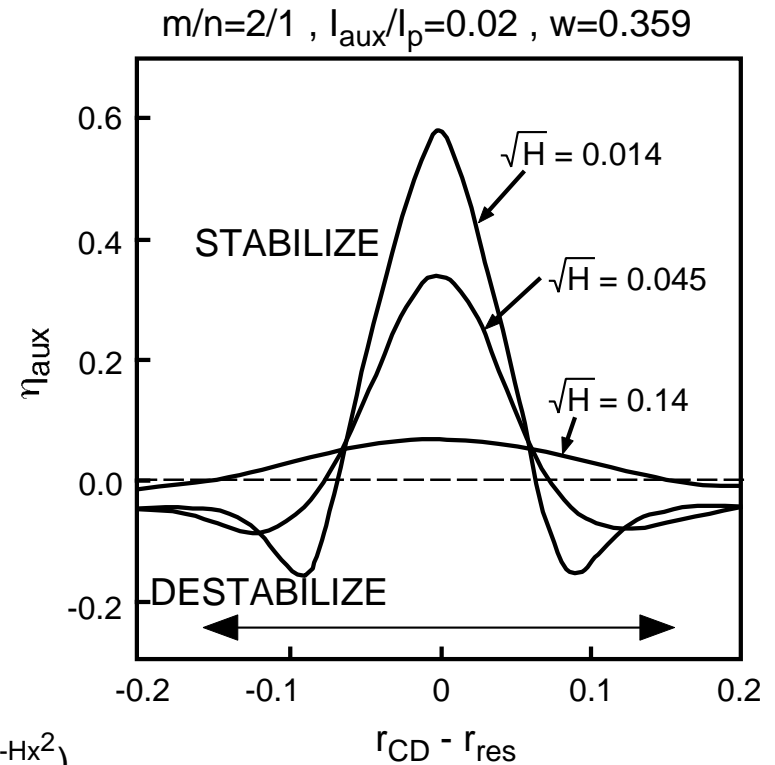
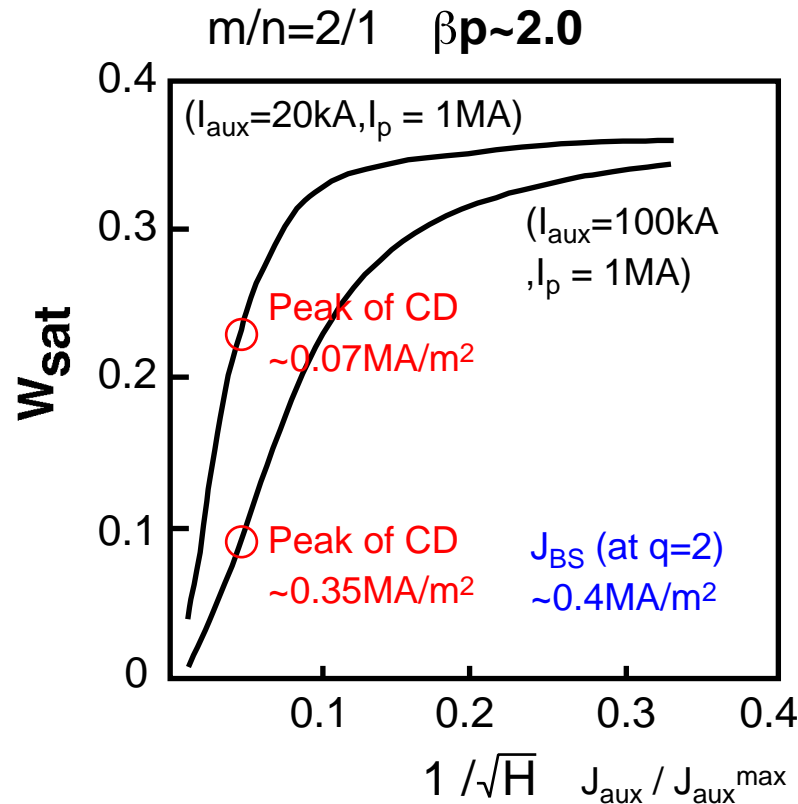
Stabilization of NTM

JT-60SC

- $\beta N \sim 3.2$, $q_0 \sim 1.2$, Broad Pressure (A), low $l_i \sim 0.8$



ECCD is not modulated to the island.



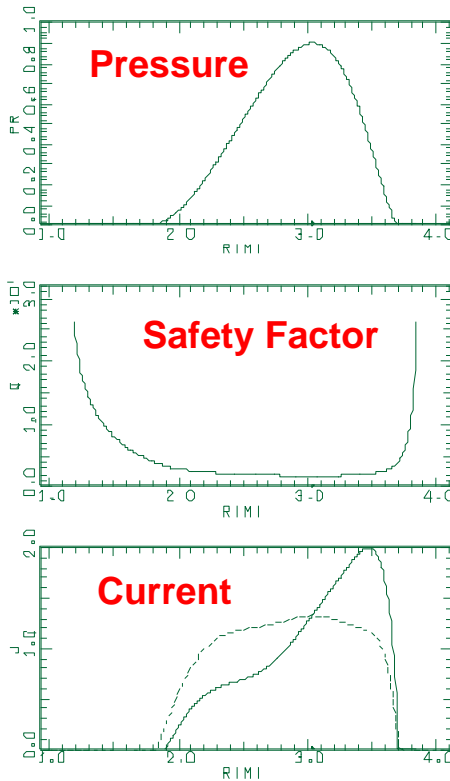
Stabilization effect is sensitive to the relative location.

- When current drive efficiency is ~ 0.05 near $r=a/2$, $R_0=0.28m$, $n_{e0}=5 \times 10^{19} m^{-3}$, EC power of 4MW drives the local current of $\sim 150kA$.
- Significant stabilization of NTM by ECCD can be expected.
- Real time feedback using ECE, etc., and ECCD is key issue.

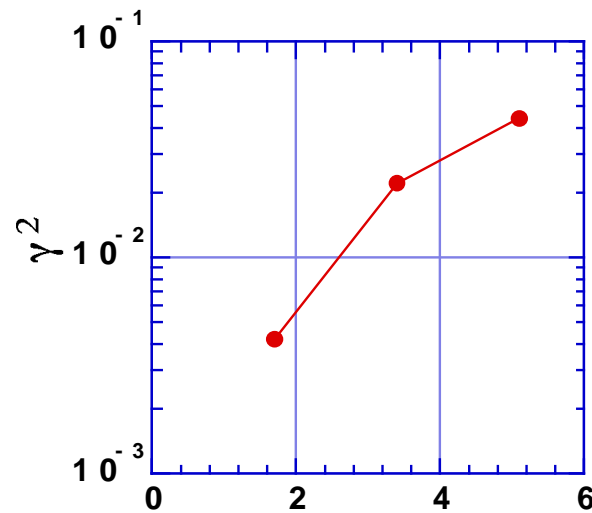
Preliminary Analysis: RWM in JT-60SC

JT-60SC

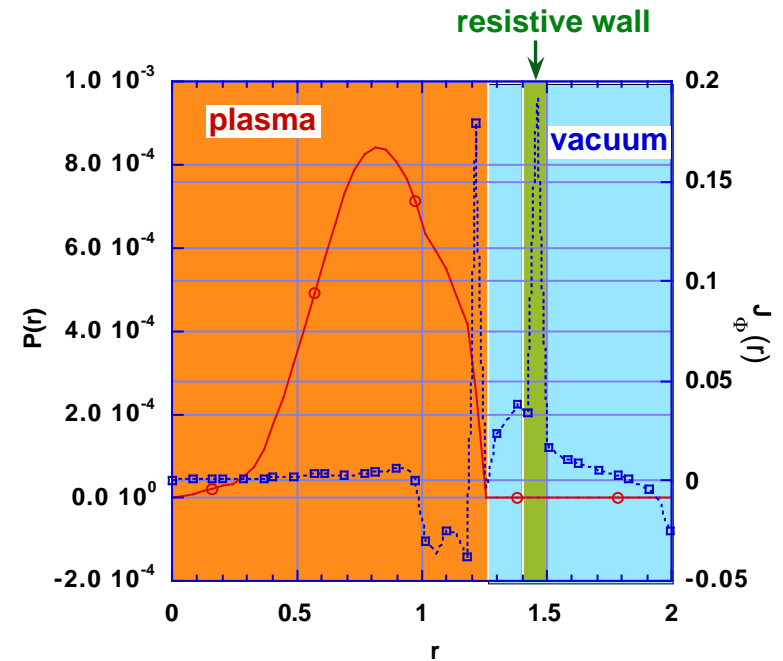
JT-60SC Equilibrium of $\beta_N=3.4$



Resistive MHD calculation with resistive wall effects. (full set MHD model)
Vacuum is represented by pseudo-vacuum occupied by high resistive plasma



Growth Rate



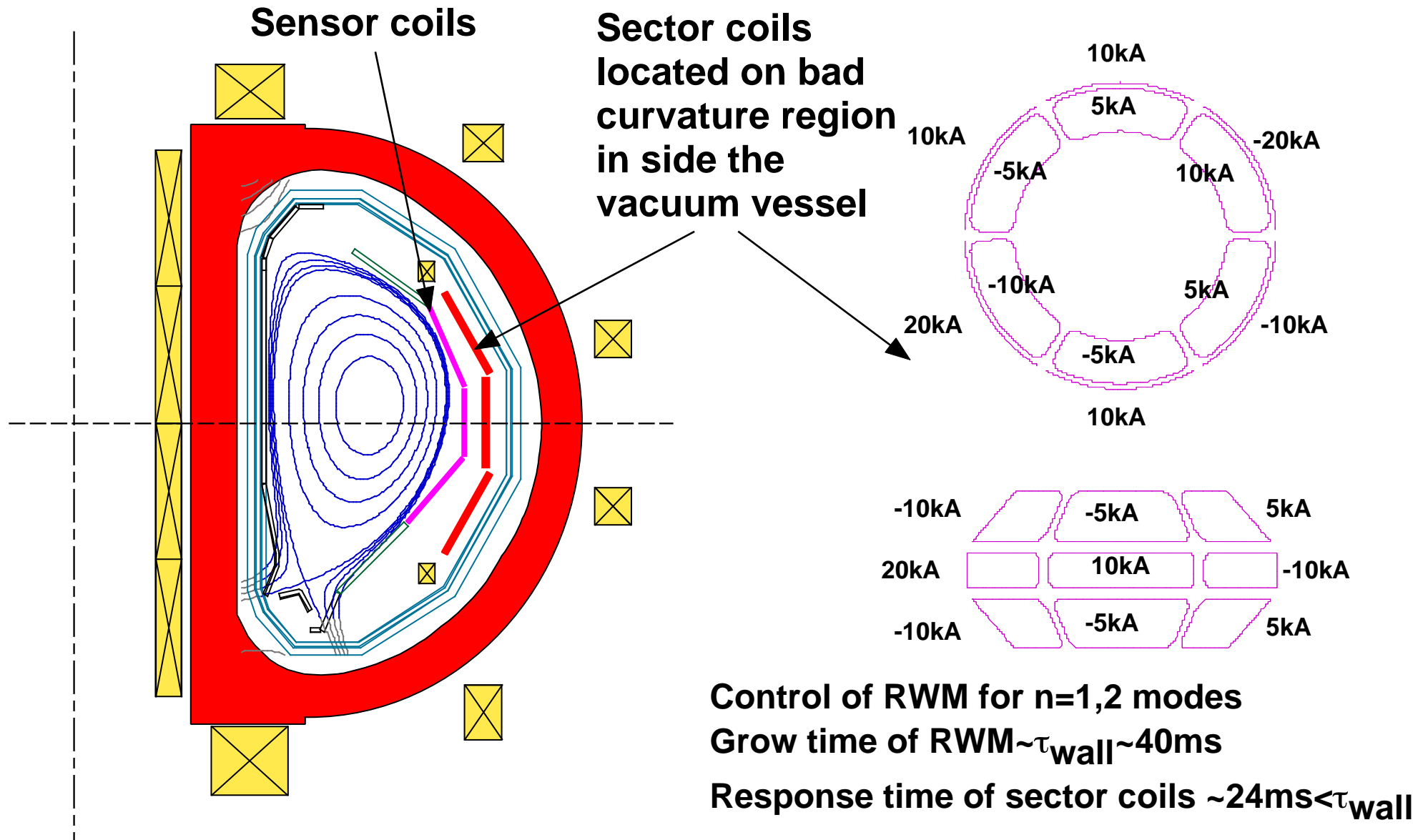
Eigen Function (m/n=4/1)

$r_{\text{wall}} / r = 1.16$

Destabilization of RWM is confirmed.

Tool of RWM Stabilization

JT-60SC



Issue: Algorithm of real time feedback for stabilization

Neutral Point Analysis by TSC Simulation

JT-60SC

- Vertical Displacement Event (VDE) induced during disruptions is suppressed by the neutral point.

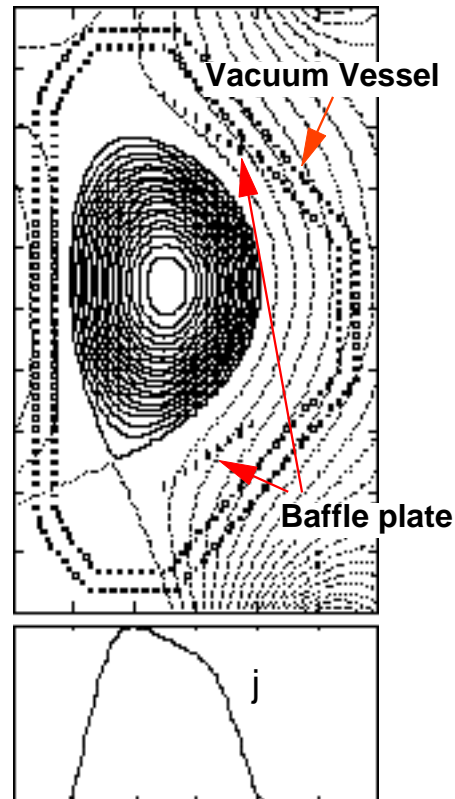
• TSC Scimulation: Standard Configuration of JT60-SC

Plasma

$$I_p = 3.0 \text{ MA,}$$
$$(R_{\text{mag}}, Z_{\text{mag}}) = (2.76 \text{ m, } 0.20 \text{ m}),$$
$$I_i = 0.97, \beta_p = 0.001,$$
$$\kappa = 1.68, \delta = 0.43,$$
$$\text{Vol} = 51.9 \text{ m}^3,$$
$$n\text{-index} = -1.3$$

Resistive Shell

Double-Walled Vacuum Vessel +
Up-Down Asymmetric Baffle Plate



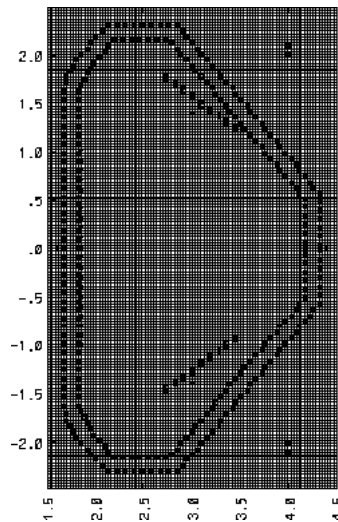
- I_p -quench rate = $\sim 21 \text{ msec}^{-1}$ (\sim eddy current decay rate of vacuum vessel)
- Baffle plate of JT60-SC is positioned upward by + 15 cm.
- Without any of active feedback
- Vertical shifts (dZ) were measured at 3 msec after the start of I_p -quench.

Neutral Point of JT-60SC

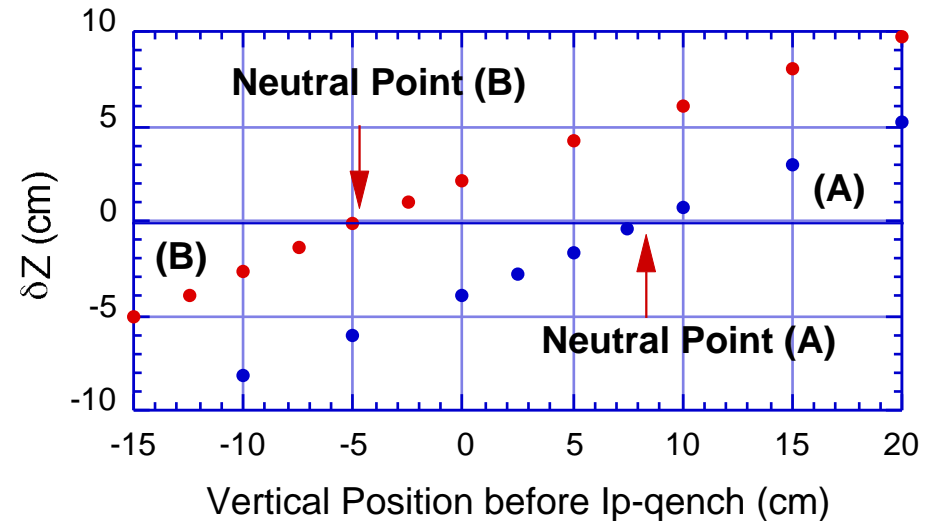
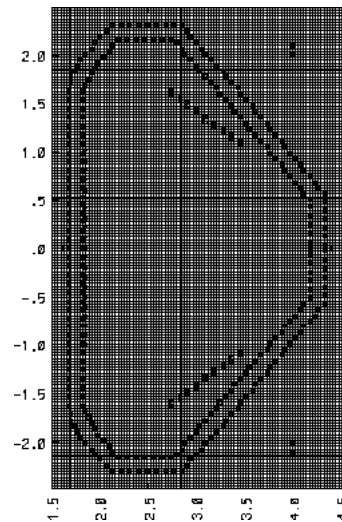
JT-60SC

- **Neutral Point of JT60-SC**

**(A) Up-Down
Asym. Baffle Plate**



**(B) Up-Down
Sym. Baffle Plate**

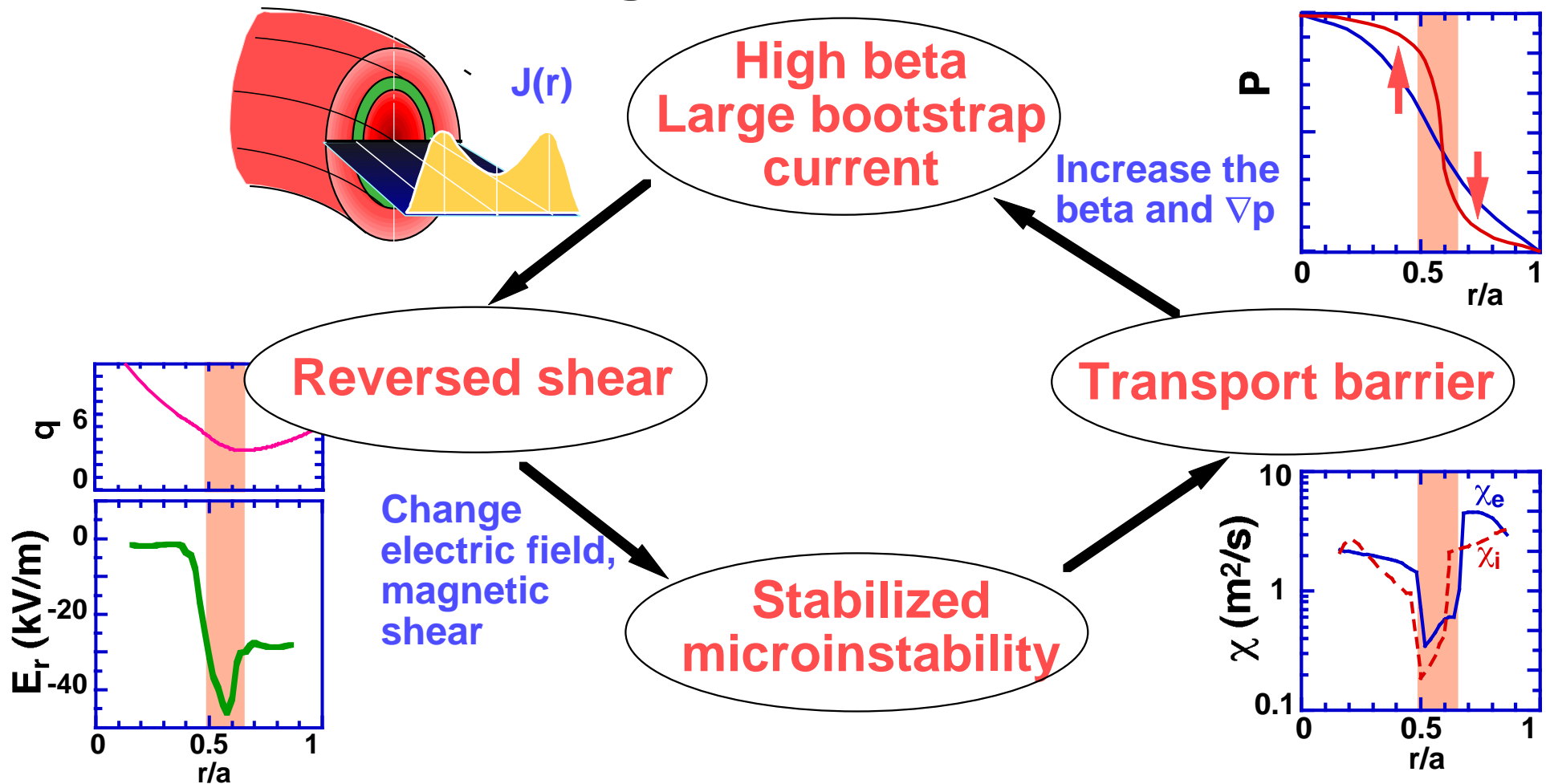


Baffle plate mainly affects the vertical location of neutral point in JT60-SC.

- **Neutral Point exists at ~ 8.0 cm above JT60-SC midplane.**
(~ -12.0 cm below the plasma magnetic axis of the standard configuration)
- **However, optimization of baffle plate design is required.**
- **Effects of plasma shape, current profile, equilibrium field structure on the location of Neutral Point are now under investigation.**

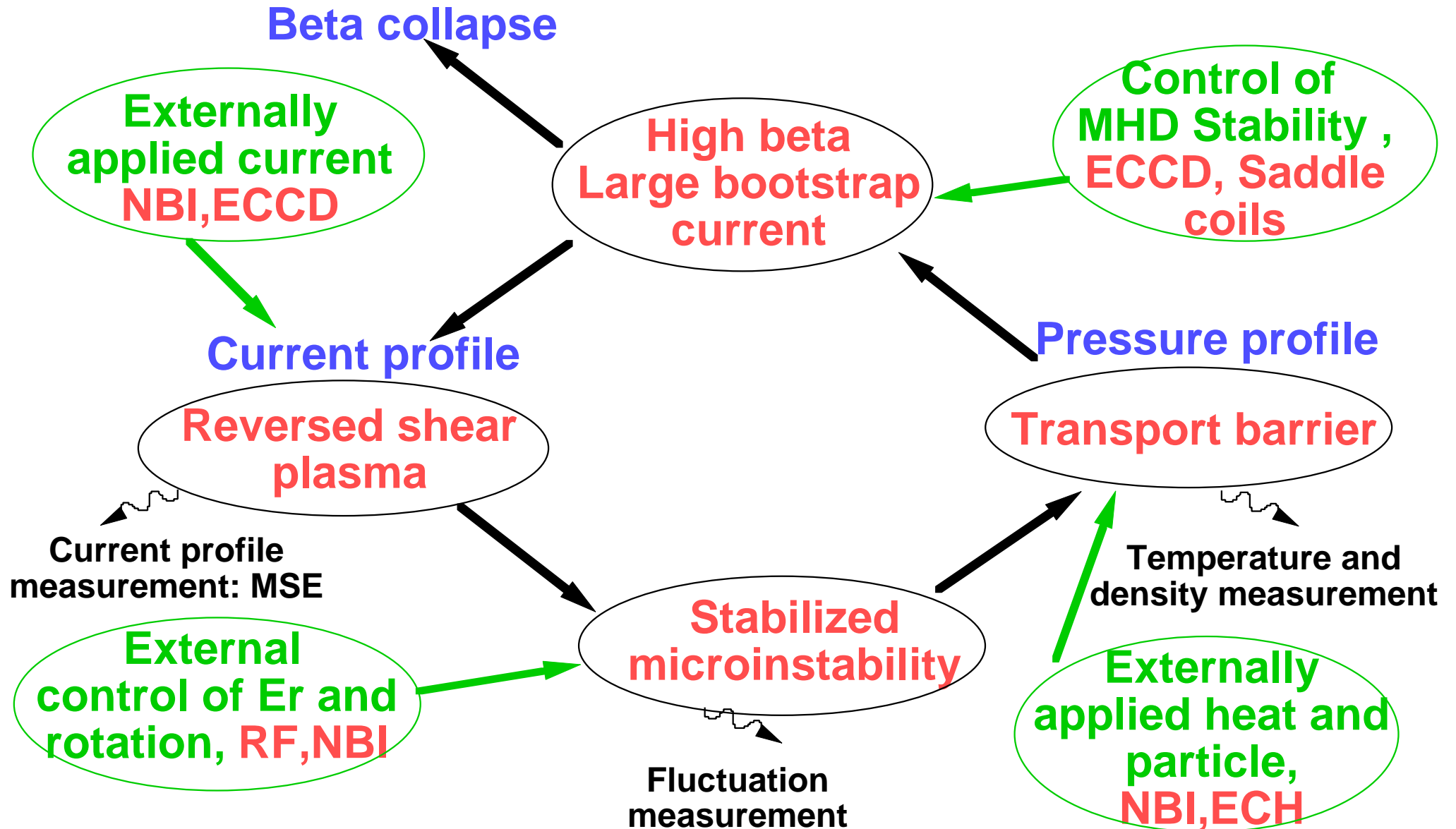
Physics Issue of High beta and steady-state plasma

"Intrinsic Positive Feedback" in the High Bootstrap Plasma



Control of "Intrinsic Positive Feedback"

JT-60SC



Can we obtain the good confinement with ITB in beta plasma, steadily?

JT-60SC

- In low beta plasma with low bootstrap current, correlations of each agent (element phenomena) is weak .

Answer is Yes.

- In the steady state and high beta plasma, correlation of each agent with nonlinear characteristics is strong, which makes the chaos and the complexity.

Answer is question.

Summary

Establish high beta in steady-state and high Bt plasma by

- utilization of high power and flexible NBI system, and ECH/ECCD,**
- using the active feedback system of ECCD and saddle coils.**

Exploit the new physics for steady-state plasma

- Nonlinear phenomena**
- Chaos**
- Complexity**